

(Status of) The ITER project

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Outline

- **What is ITER? \Rightarrow main physics and technology goals**
- **Status of the site**
- **Device dimensions and operational regimes**
- **Design of key components**

ITER is a major (€15bn) collaboration in fusion energy research by seven international partners

- **Partners:** EU (+ Switzerland), China, India, Japan, Russian Federation, South Korea, USA

INTERNATIONAL CONTRIBUTORS TO ITER



- **Overall program objective:** demonstrate the scientific and technological feasibility of fusion energy for peaceful purposes
- **Principal goal:** design, construct and operate a tokamak experiment at a scale satisfying objectives

ITER is a major (€15bn) collaboration in fusion energy research by seven international partners

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ITER is designed as a burning (D-T) plasma experiment with significant α -particle heating

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INTERNATIONAL
CONTRIBUTORS TO ITER



ITER's project challenges are significant

- **Unprecedented size and technology:** large and heavy components, challenging to be built by industry (one of a kind pieces)
- **Highly integrated components, built in different parts of the world:** quality assurance, machine interface, integration
- **Total cost is approx. 15 bn Euros:** political issue (in particular during current economic turndown), projects cap/management
- **Long time scales, including 10 years of construction and 20 years of operation:** political commitment, maintenance periods are expected to be lengthy and difficult

ITER's project challenges are significant

- Unprecedented size and technology: “uncomfortably”

ITER is primarily an experiment, but also a major technology test bed



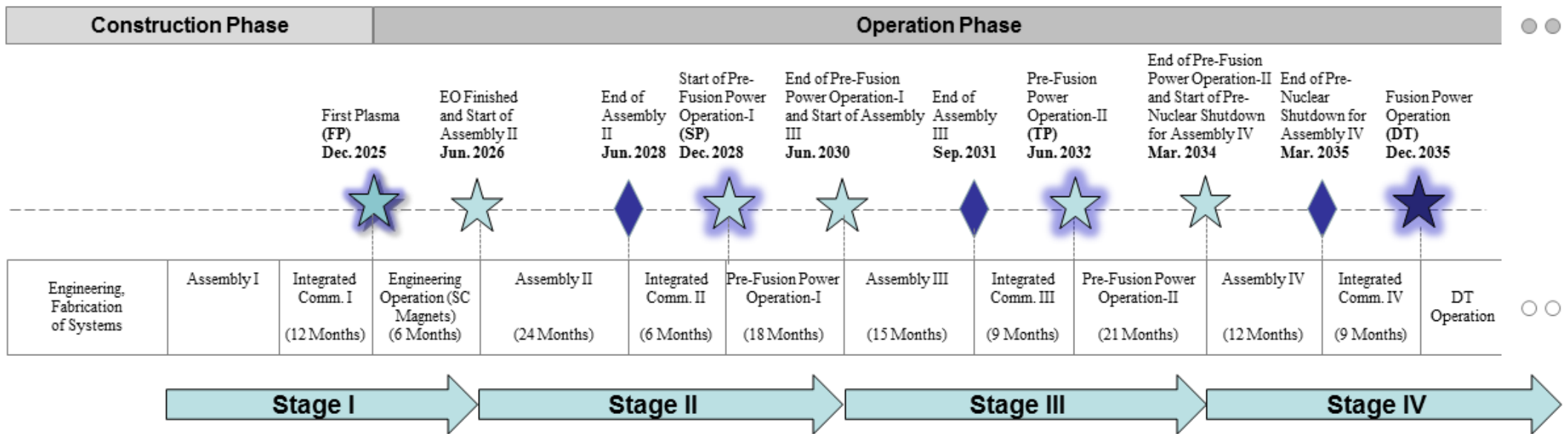
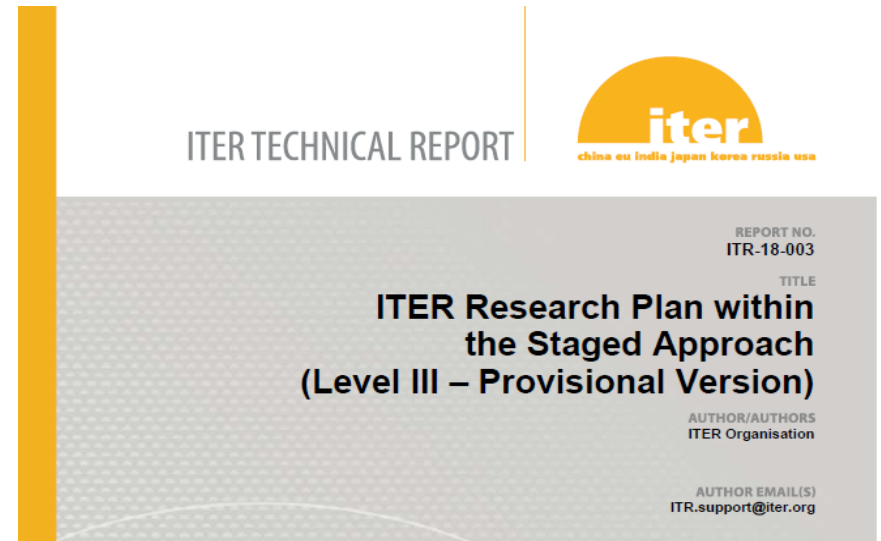
Essential step toward DEMO

(or demonstration that tokamaks do not work)

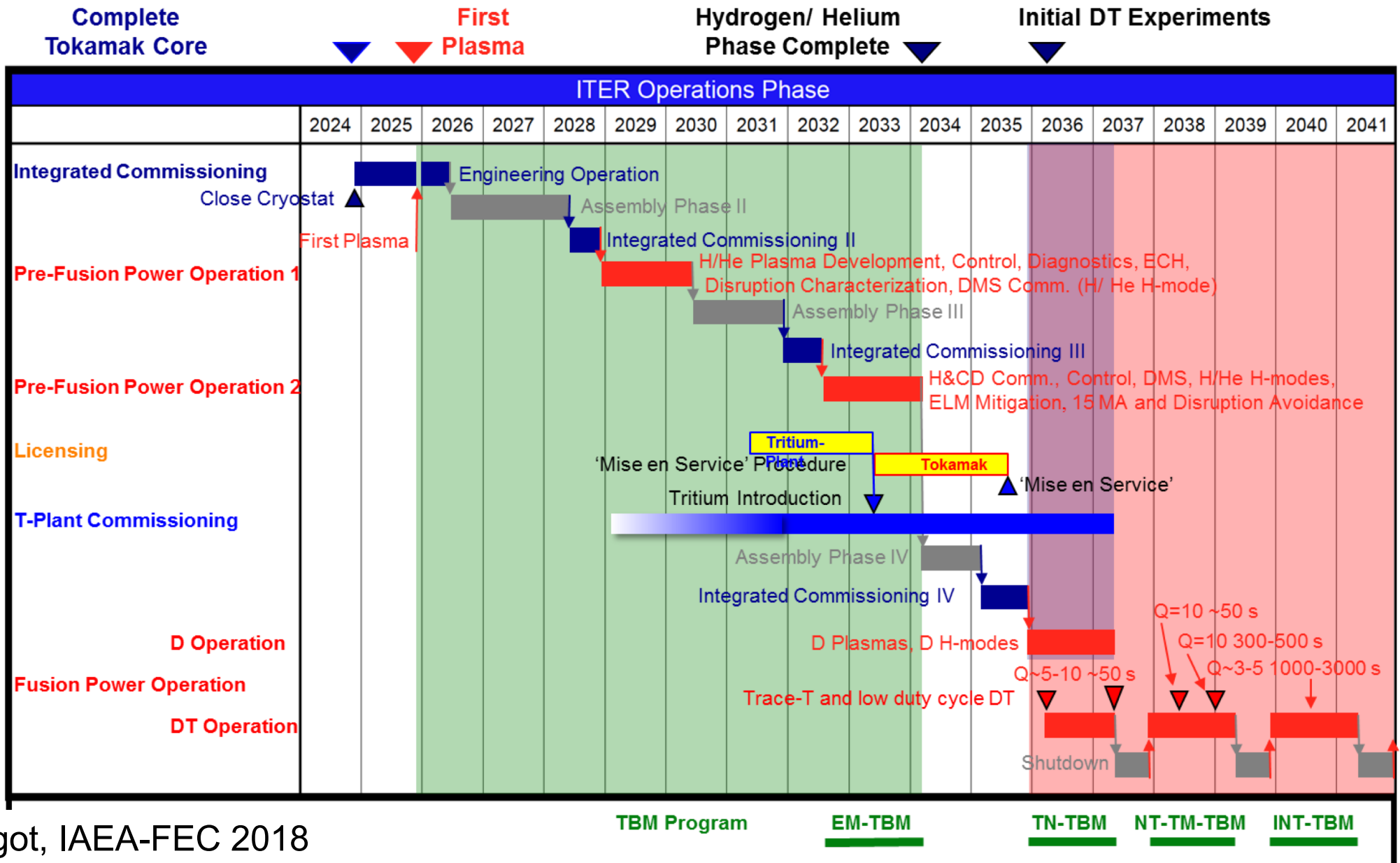
- Highly integrated components, built in different parts of the world: quality assurance, machine interface

A revised version in 2016 put the start of ITER operation in December 2025, and D-T in Dec 2035

- ITER Research Plan publicly available as technical report (ITR-18-003):
<https://www.iter.org/technical-reports>
- Provides definition of required R&D

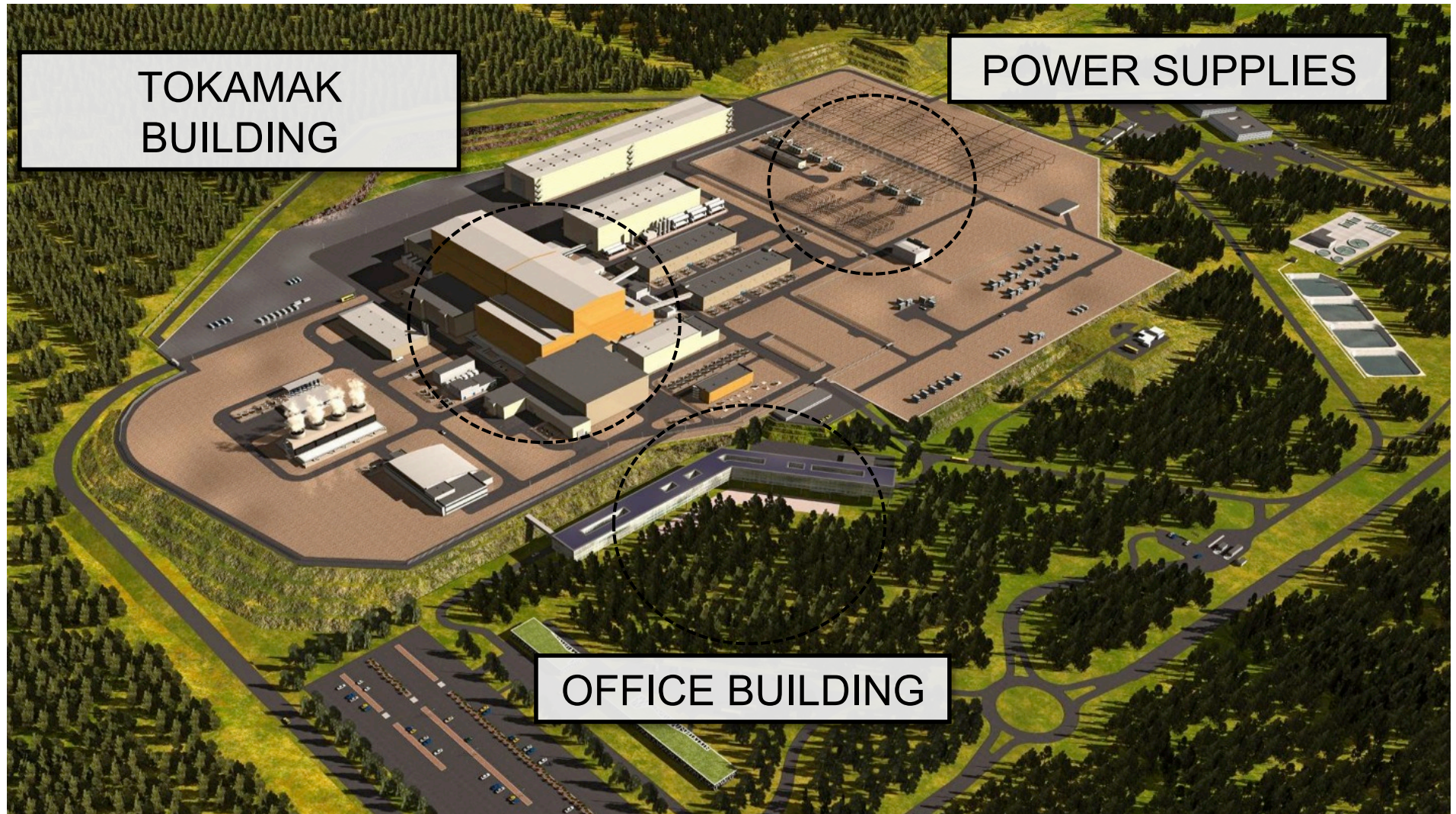


A revised version in 2016 put the start of ITER operation in December 2025, first DT ops in 2036



Bigot, IAEA-FEC 2018

The ITER site includes a total of 39 buildings on 180 hectares



The ITER site in 2005: a woodsy area in southern France

2005



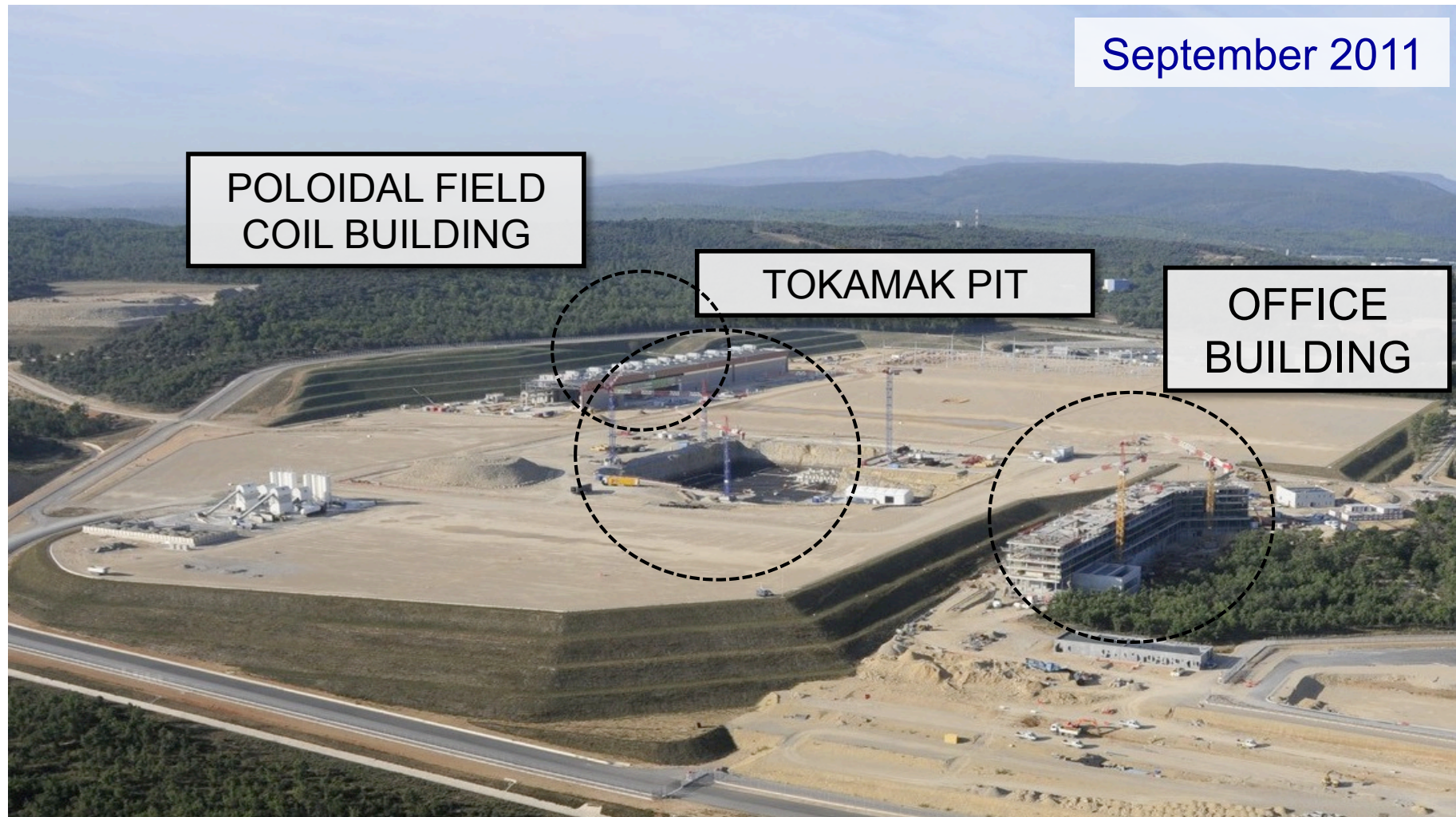
Site preparation was finished in 2009 over a 4-year period

40 ha platform, 2.5 million m³ of earth moved, good bedrock (100 t m⁻²)



2009

The construction of the ITER buildings commenced in September 2011

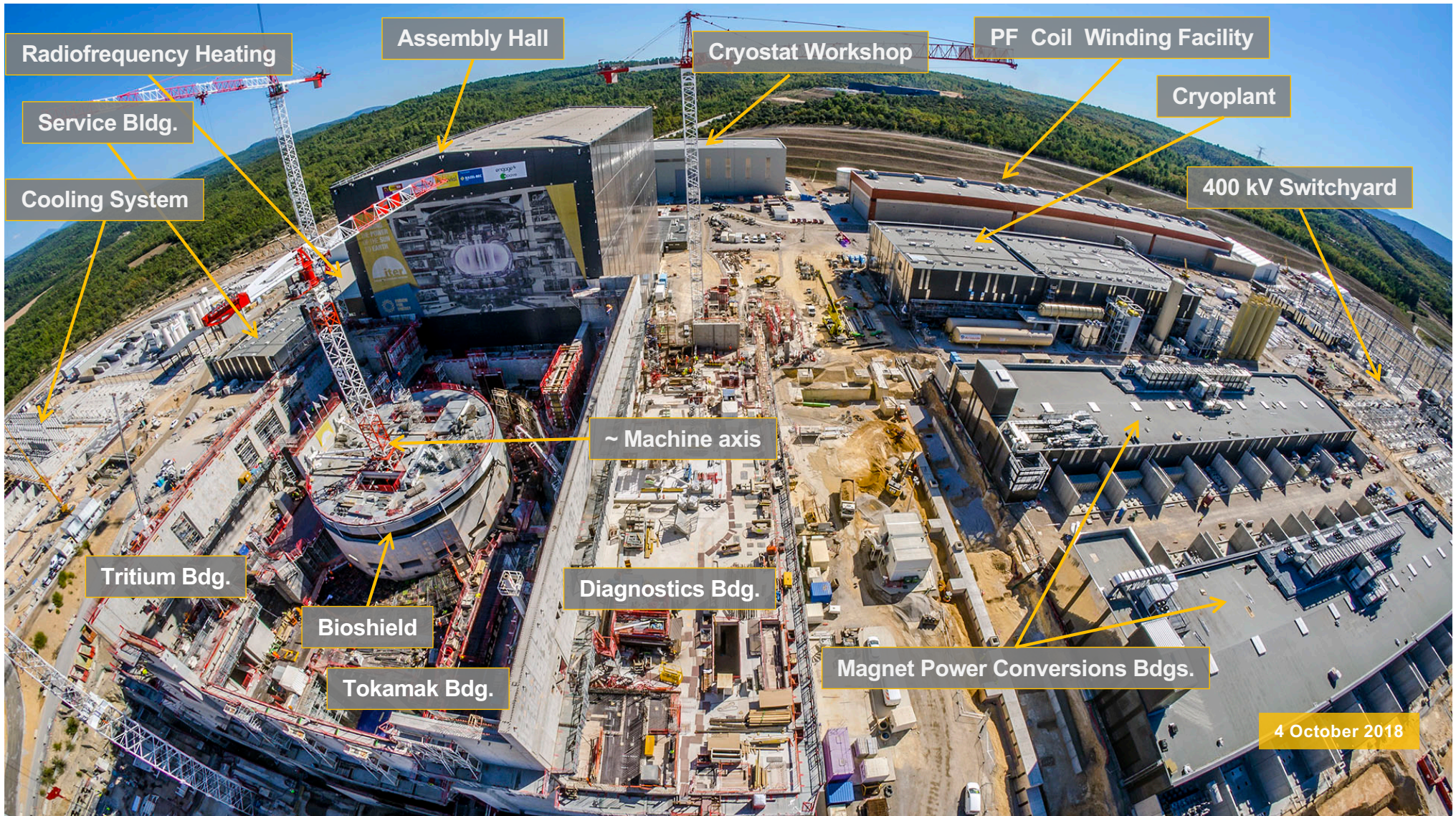


Aerial view of the site in September 2013

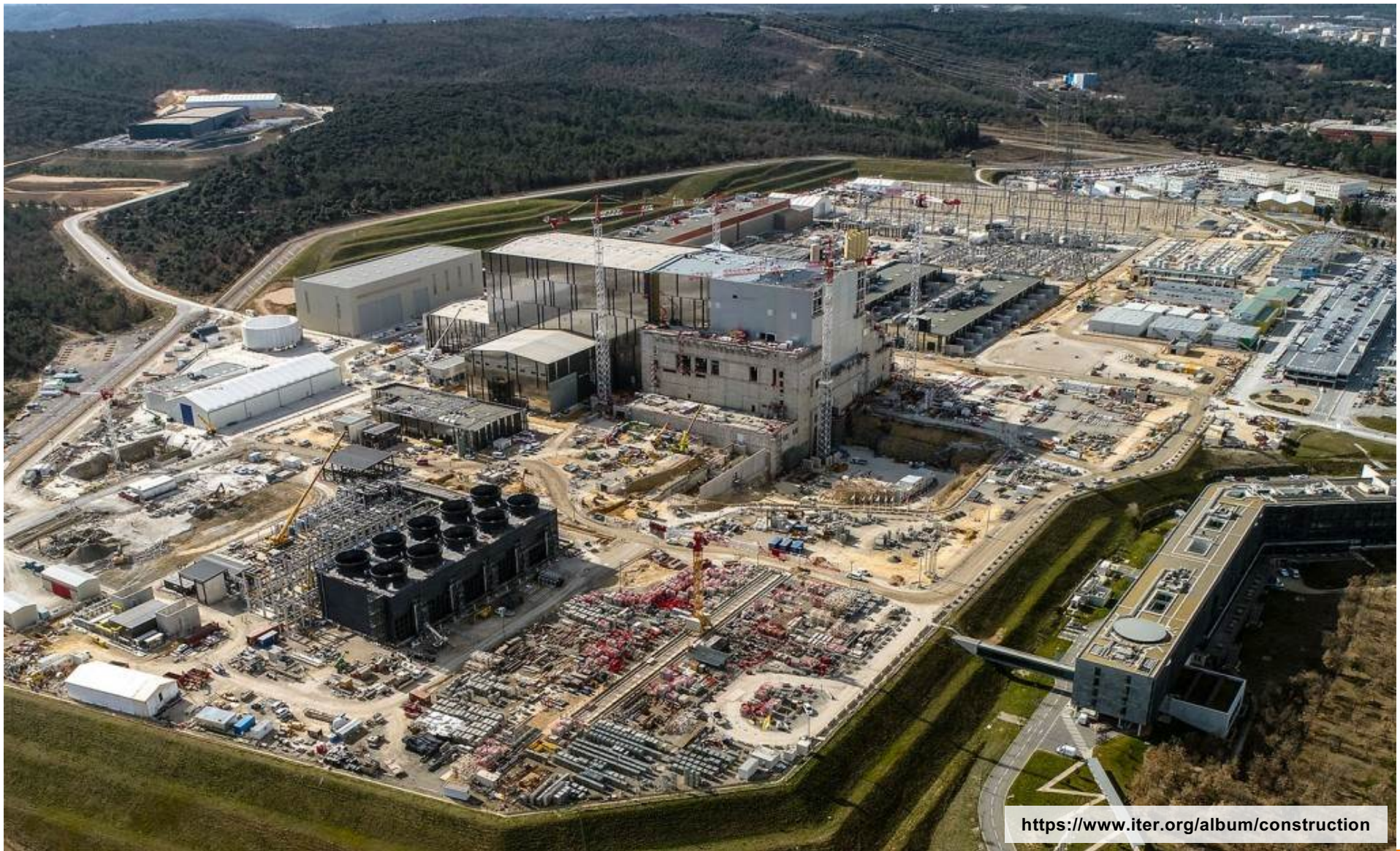
September 2013



The ITER work site on Oct 4, 2018



The ITER work site on February 2020



**Progress on the work site Nov 2014 versus Nov 2020:
75% of the civil engineering work is completed**

Six years of steady progress

Bigot, Fusion Power Associates, Dec 2020



November 2014



November 2020

More than 75% of the installation's civil works are now completed.

The ITER work site on November 2020

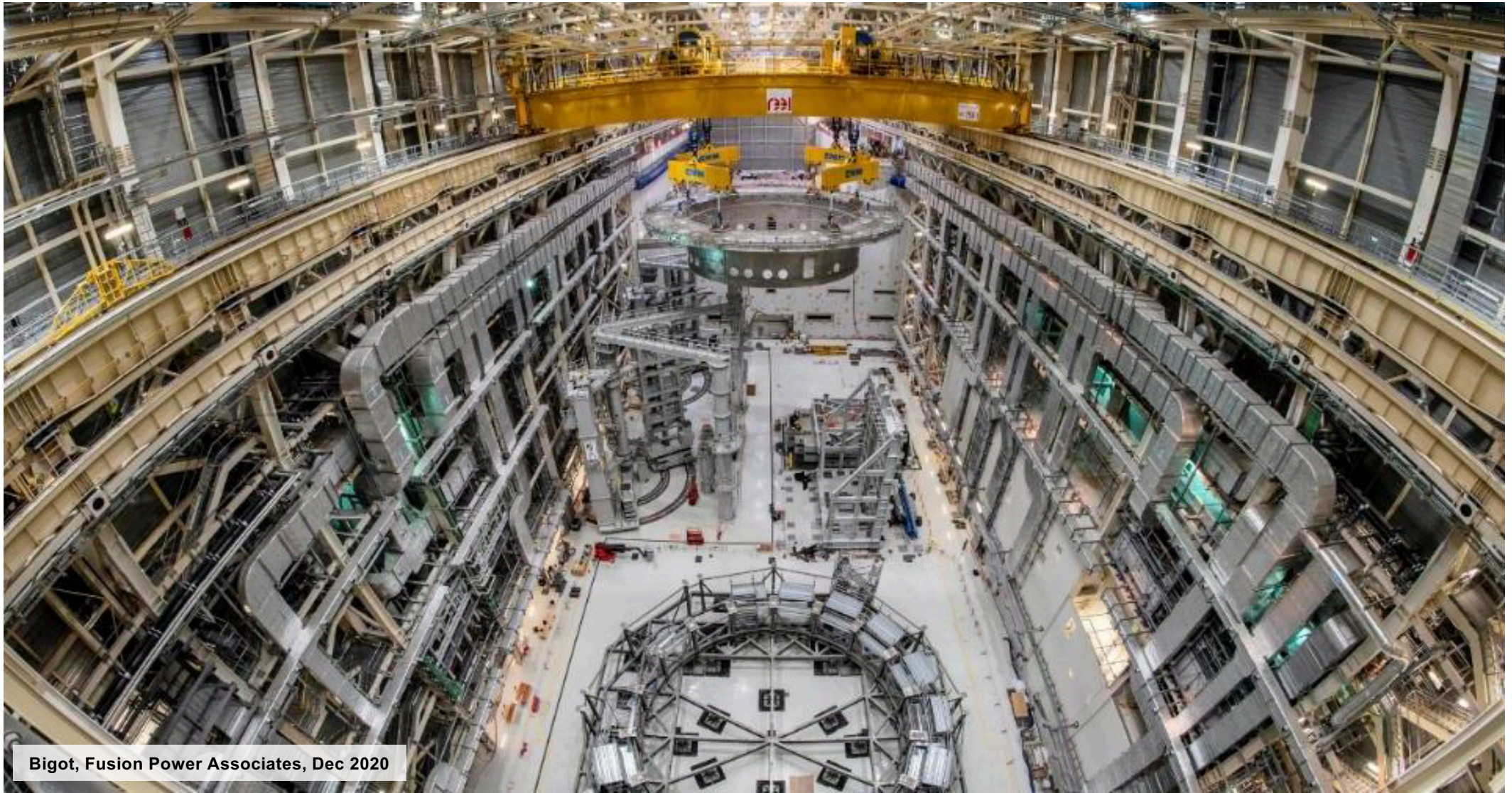


Bigot, Fusion Power Associates, Dec 2020

Site construction June 2020: ITER by drone

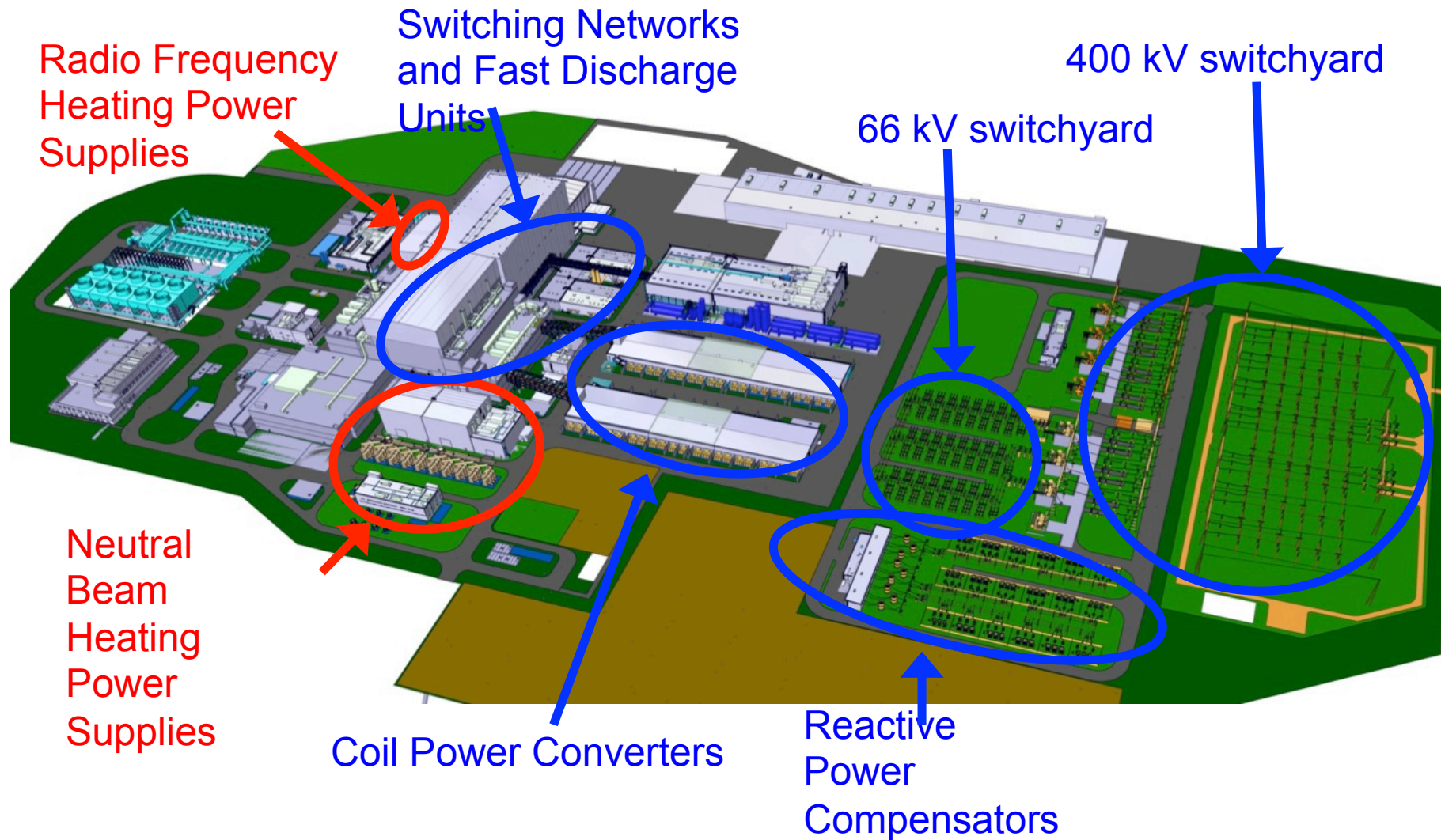
https://www.youtube.com/watch?v=_7-pvJ8IRBg

In May 2020, the base of the Cryostat (1,250 t) was successfully inserted into the tokamak assembly pit

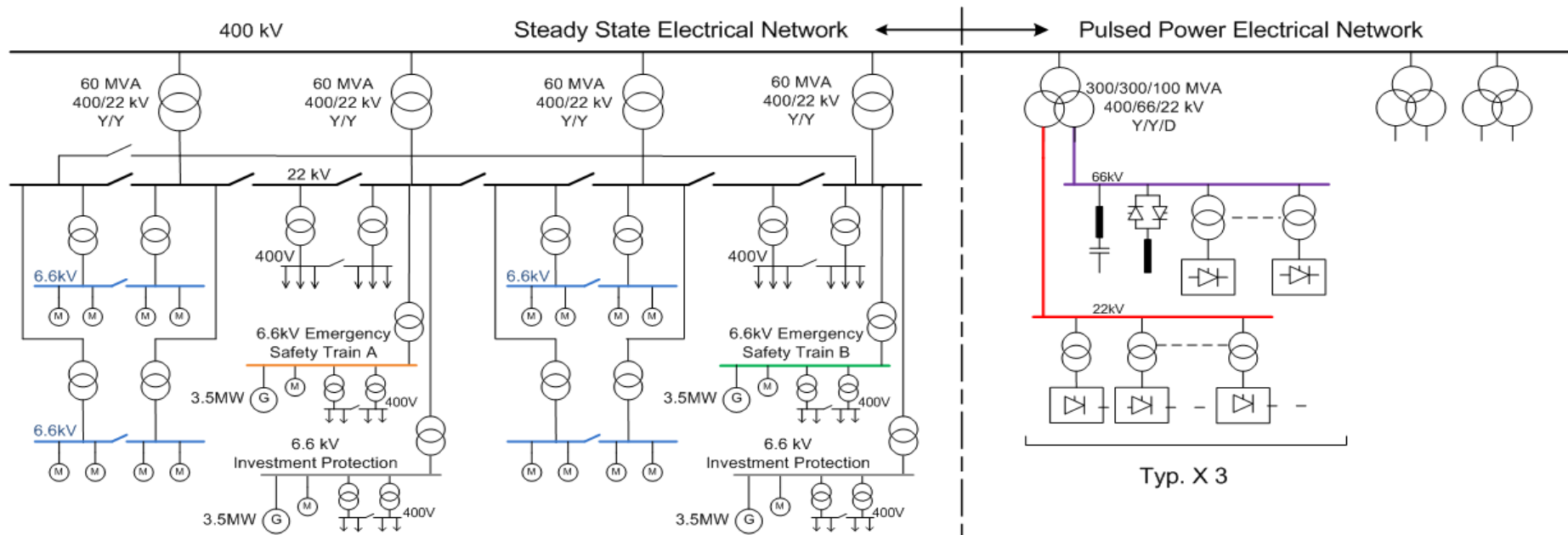


Bigot, Fusion Power Associates, Dec 2020

The 400 kV (and 66 kV) switch yards serve the tokamak and auxiliary heating systems



The pulsed electrical network can deliver about 500 MW peak pulse



Steady State Electrical Network

about 120 MW continuous power

Main consumers:

- Cooling Water System
 - Cryoplant
- Building services

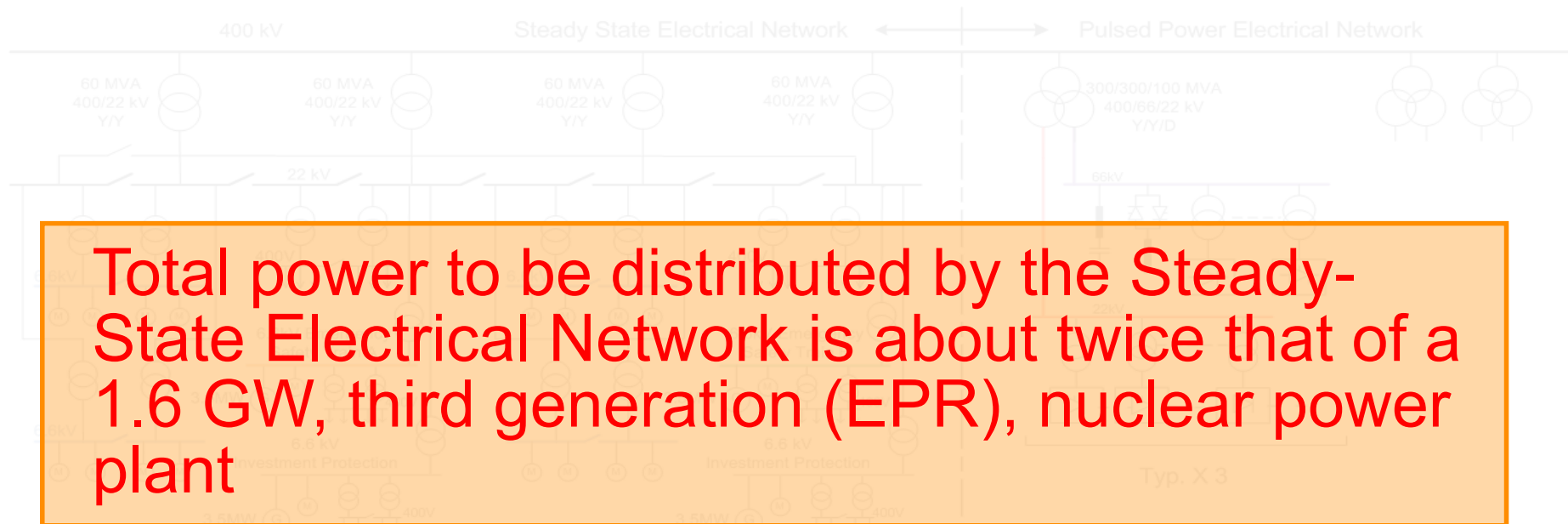
Pulsed Power Electrical Network

about 500 MW peak pulse

Main consumers:

- Coil power converters
 - Radio Freq. and Neutral Beam systems
- Includes large Static Var Compensators

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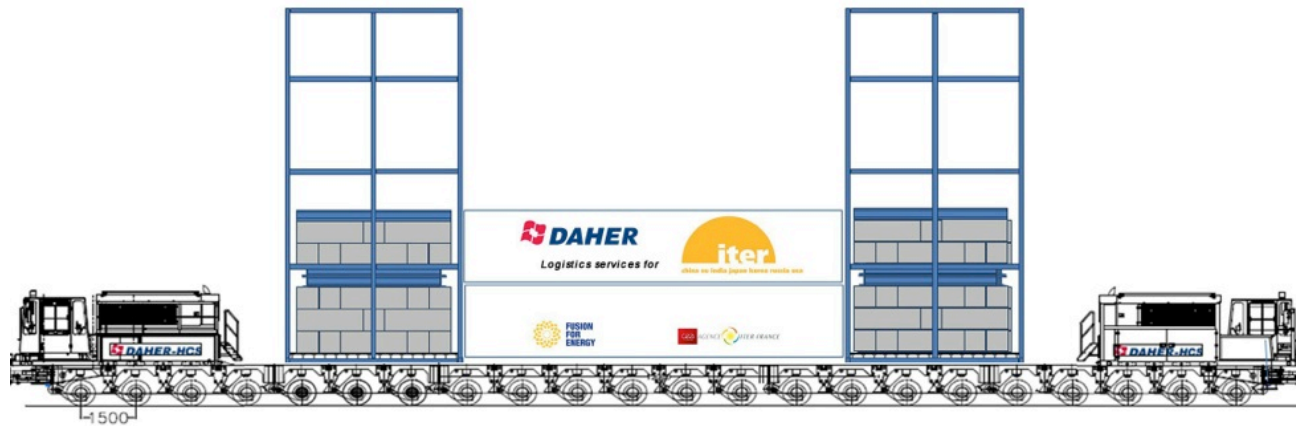
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Large components arriving in France must be transported ~100 km from the coast to Cadarache



A 352-wheel test convoy carried 800 ton load through southern France in September 2013



Arrivals of major components from offsite laboratories and factories to the ITER site in 2020

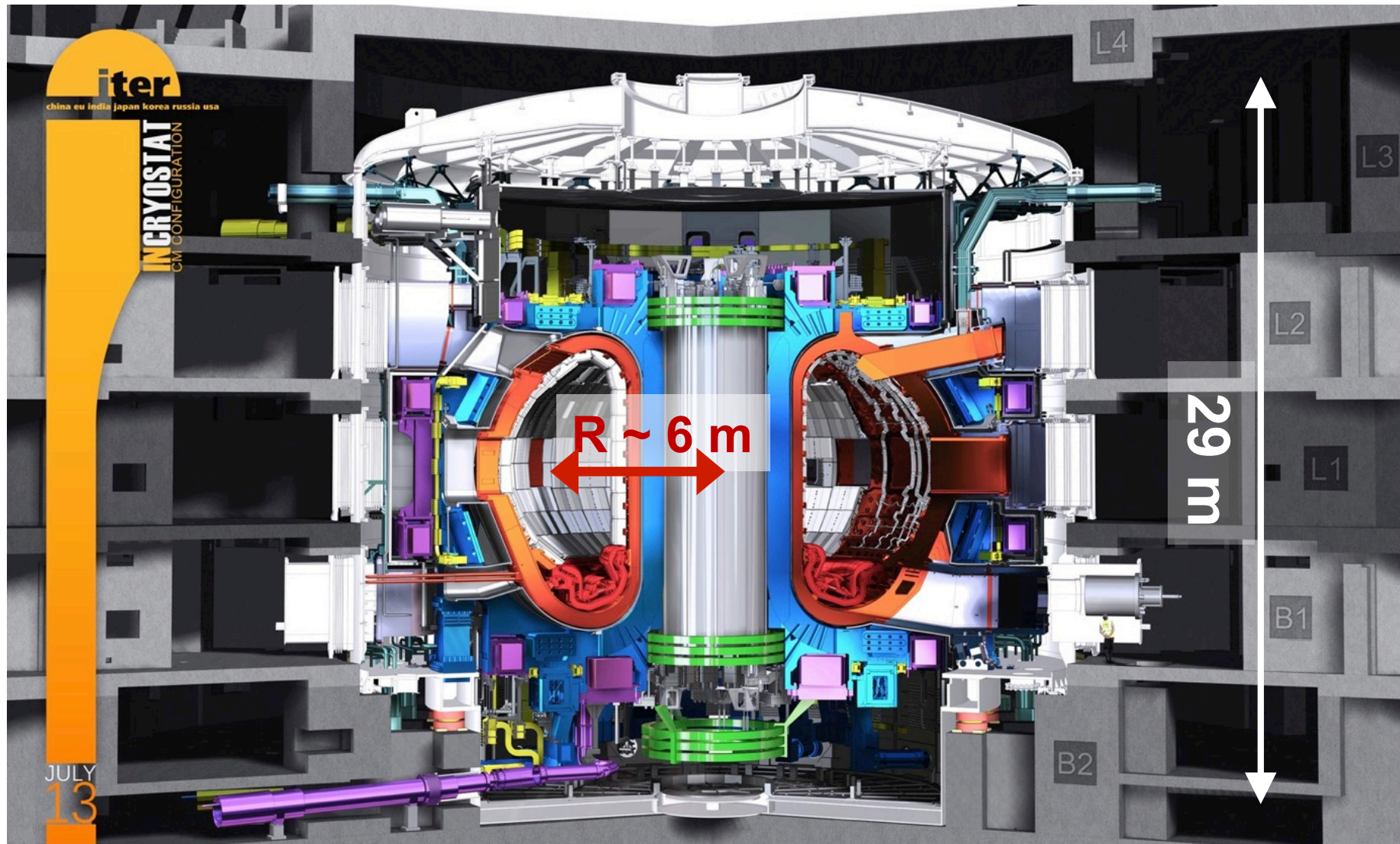


Fusion Power Associates Annual Meeting, 16 December 2020

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Bigot, Fusion Power Associates, Dec 2020

ITER is a large-scale device ($R_{\text{major}} \sim 6 \text{ m}$), with superconducting coils, and thus a (heavy) cryostat



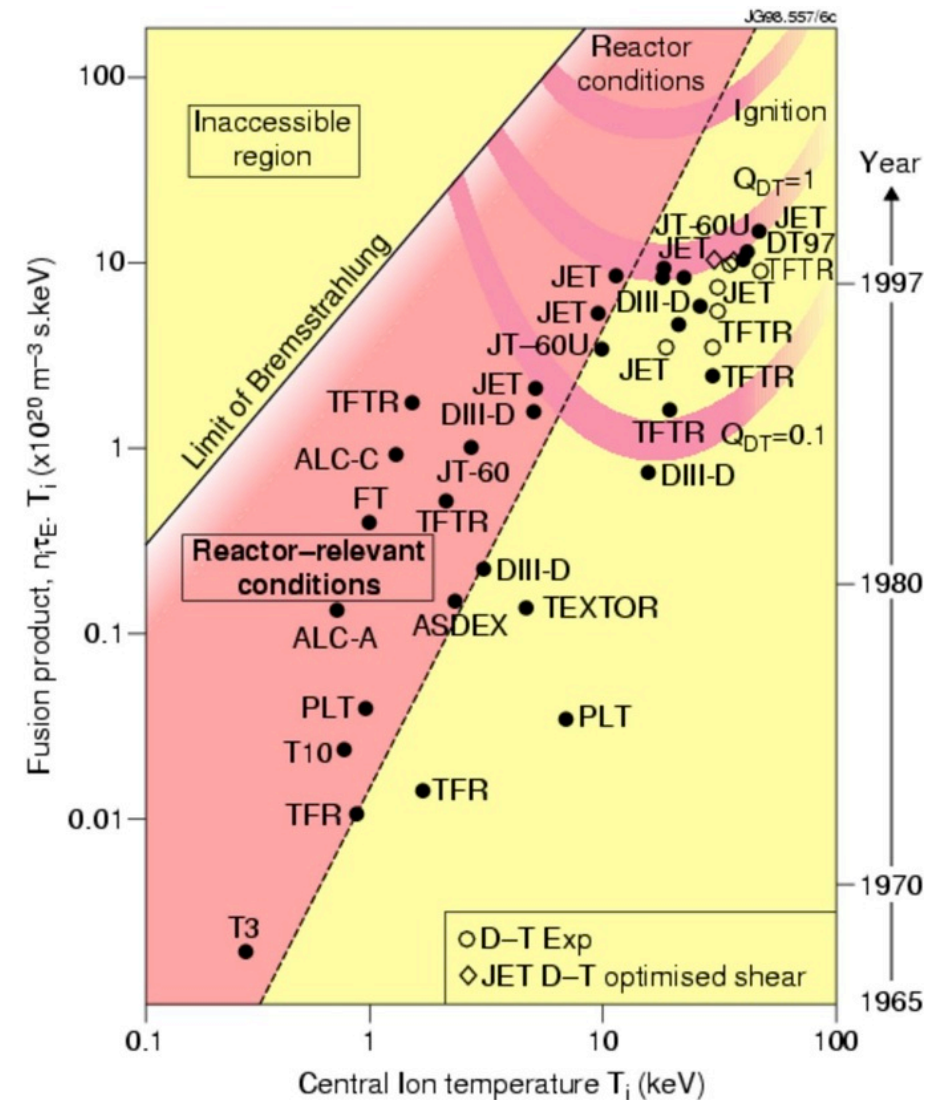
ITER is designed to integrate a burning plasma into the latest and future technology

- **Physics goals**
 - Produce significant fusion power ($Q \geq 10$) in long-pulse operations (10s of minutes)
 - Achieve full steady-state operation at moderate fusion power ($Q \sim 10$), retain possibility of exploring ‘controlled ignition’ ($Q \geq 30$)
- **Technology**
 - Demonstrate integrated operation of technology for future fusion power plant
 - Test components required for future power plants
 - Test concepts for a tritium breeding module

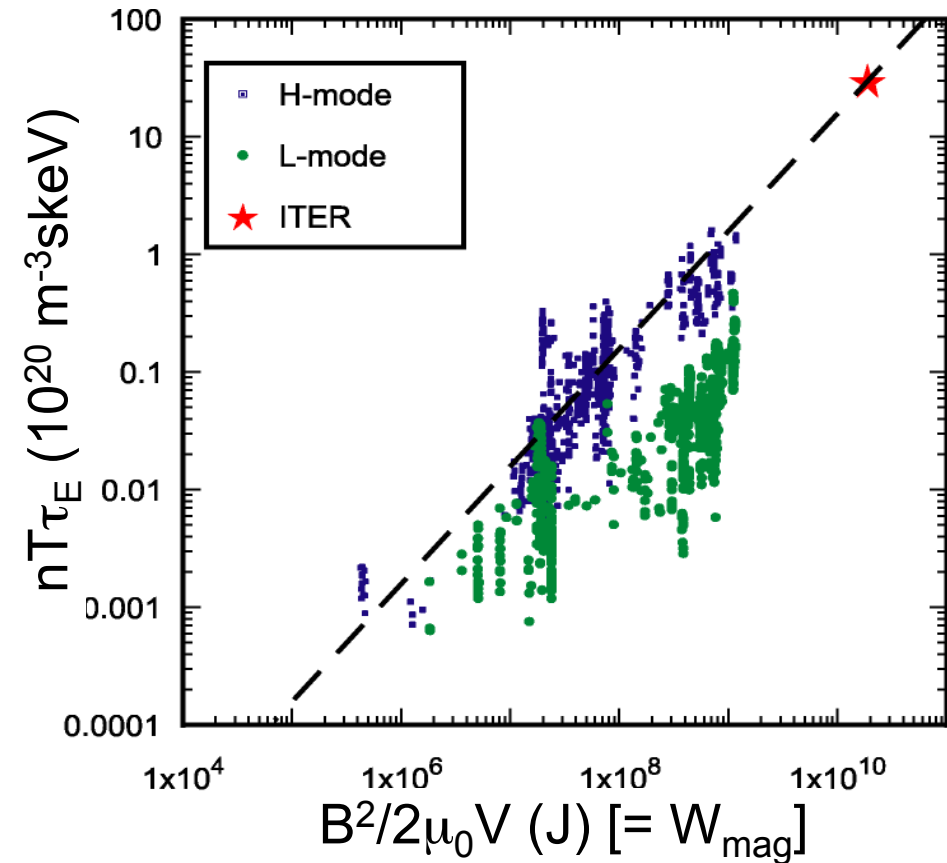
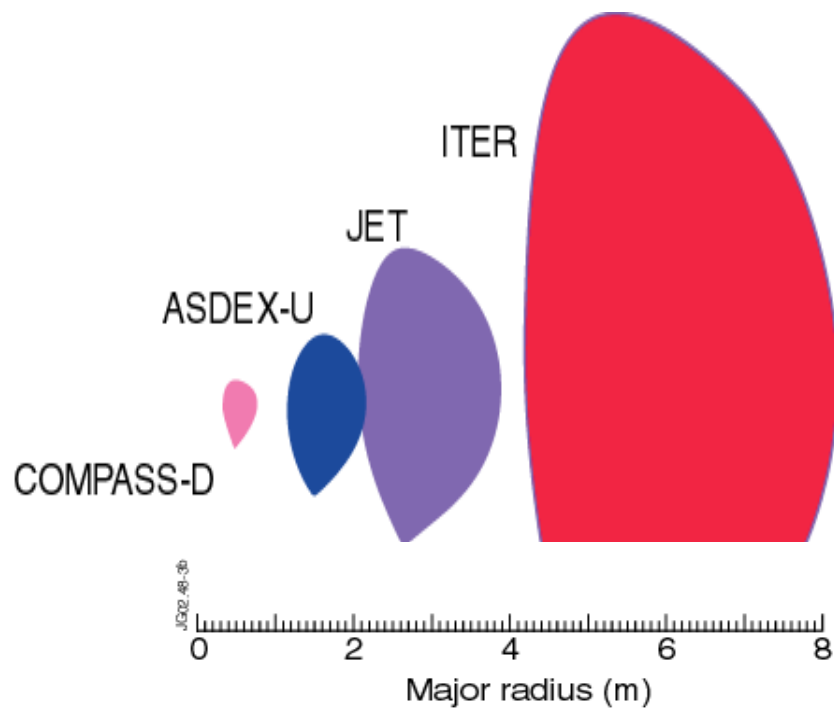
ITER is designed to yield $Q_{DT} \geq 10$ at a fusion power of up to 500 MW for up to 500 s

$$Q = \frac{\text{Fusion Power}}{\text{Input Power}} \sim n_i T_i \tau_E$$

- Existing experiments have reached $nT\tau_E \sim 1 \times 10^{21} \text{ m}^{-3} \text{ keV s}$ and $Q_{DT} \sim 1$
- JET and TFTR have produced DT fusion power **> 10 MW for 1 s**
- Various ITER reference scenarios (has to be H-mode \Rightarrow **ELMs!**)



Confinement scaling provides an approach to determine the size of ITER



- **Best performance ($nT\tau_E$) for existing devices versus stored magnetic energy is used to extrapolate to ITER**

ITER physics and operation is different to that in present-day tokamaks

- **Plasma is self-heated by fusion α -particles**
 - Non-linearity in total heating power due to dependence on plasma profiles
 - (Some) mode suppression and coupling to fast particle modes
- **ITER requires more advanced plasma control (for position, shape, fuelling, heating, stability, exhaust)**
 - Long time constant for position control (~ 1 s)
 - Complex control matrix
- **Plasma currents larger than 15 MA**
 - Generation of runaway electron beams, significant disruption forces

ITER physics and operation is different to that in present-day tokamaks

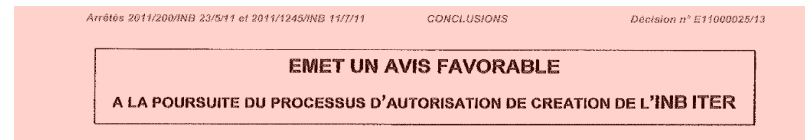
- **High ion fluence (time-integrated plasma flux) to plasma-facing components**
 - Erosion of PFCs and migration of wall material to certain (remote) areas
 - Dust formation
- **Actively cooled systems interfaced with a burning fusion plasma**

ITER physics and operation is different to that in present-day tokamaks

- **Routine operation at $Q_{DT}=10$ forces operating near/at/transiently beyond design limits**
 - Robust machine operation mandatory
- **Full nuclear operation (tritium and neutrons)**
 - Retention of tritium in and on PFCs
 - Tritium breeding (test blankets) and re-processing
 - Remote handling for 100% of in-vessel work during nuclear phase
 - Dust inventory
 - Diagnostics
 - Superconducting coil heating
 - **Licensing**

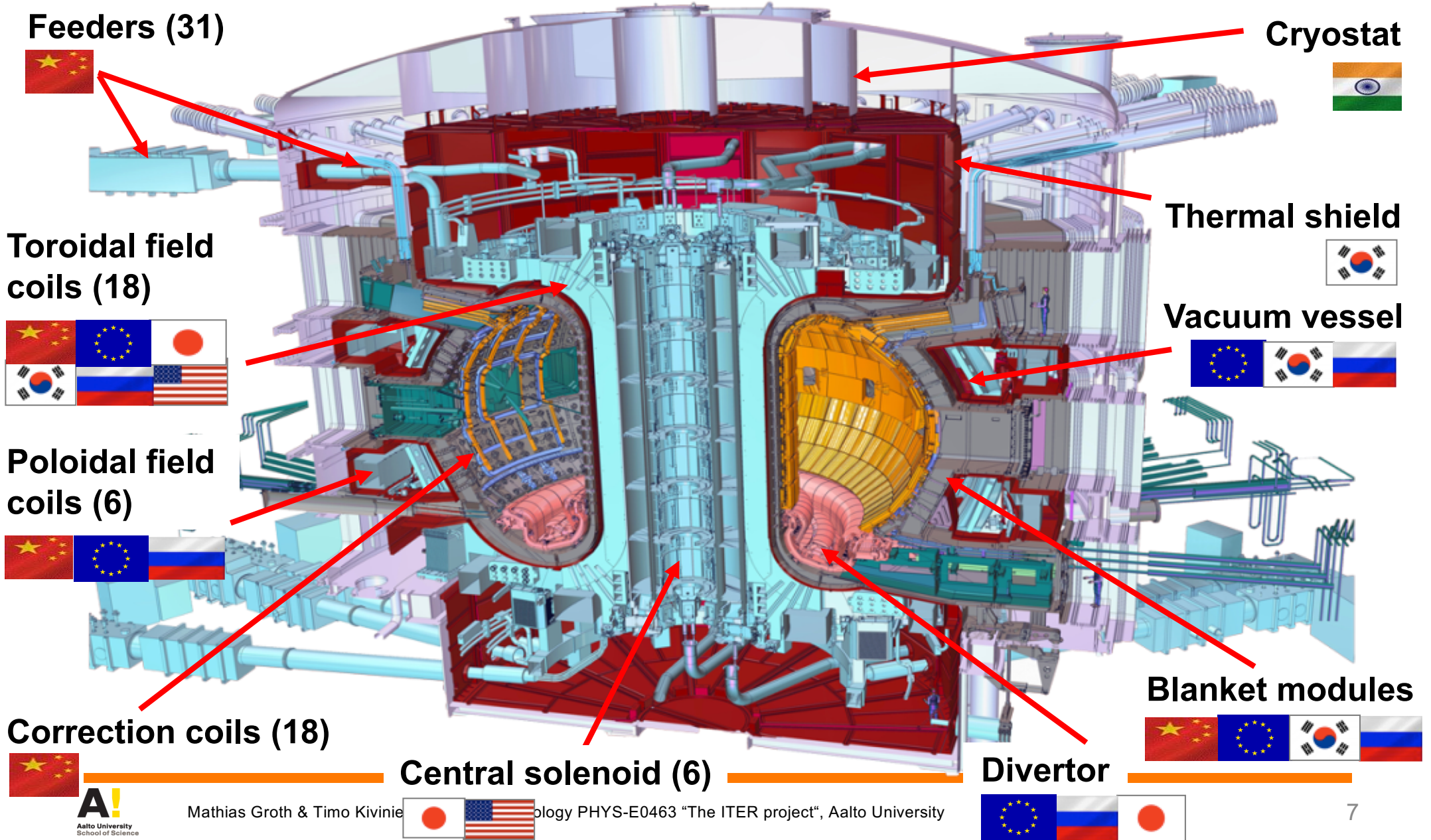
ITER is licensed as a Basic Nuclear Installation

- **ITER safety files were formally accepted by French Nuclear Authorities in December 2010**
- ⇒ **Enabled technical evaluation by the Nuclear Safety Regulator (ASN) as well as public evaluation of the system**
- **Public inquiry was conducted from June 15 to August 4, 2011 ⇒ on September 19, 2011, the Inquiry Commission issued its Advisory Opinion:**

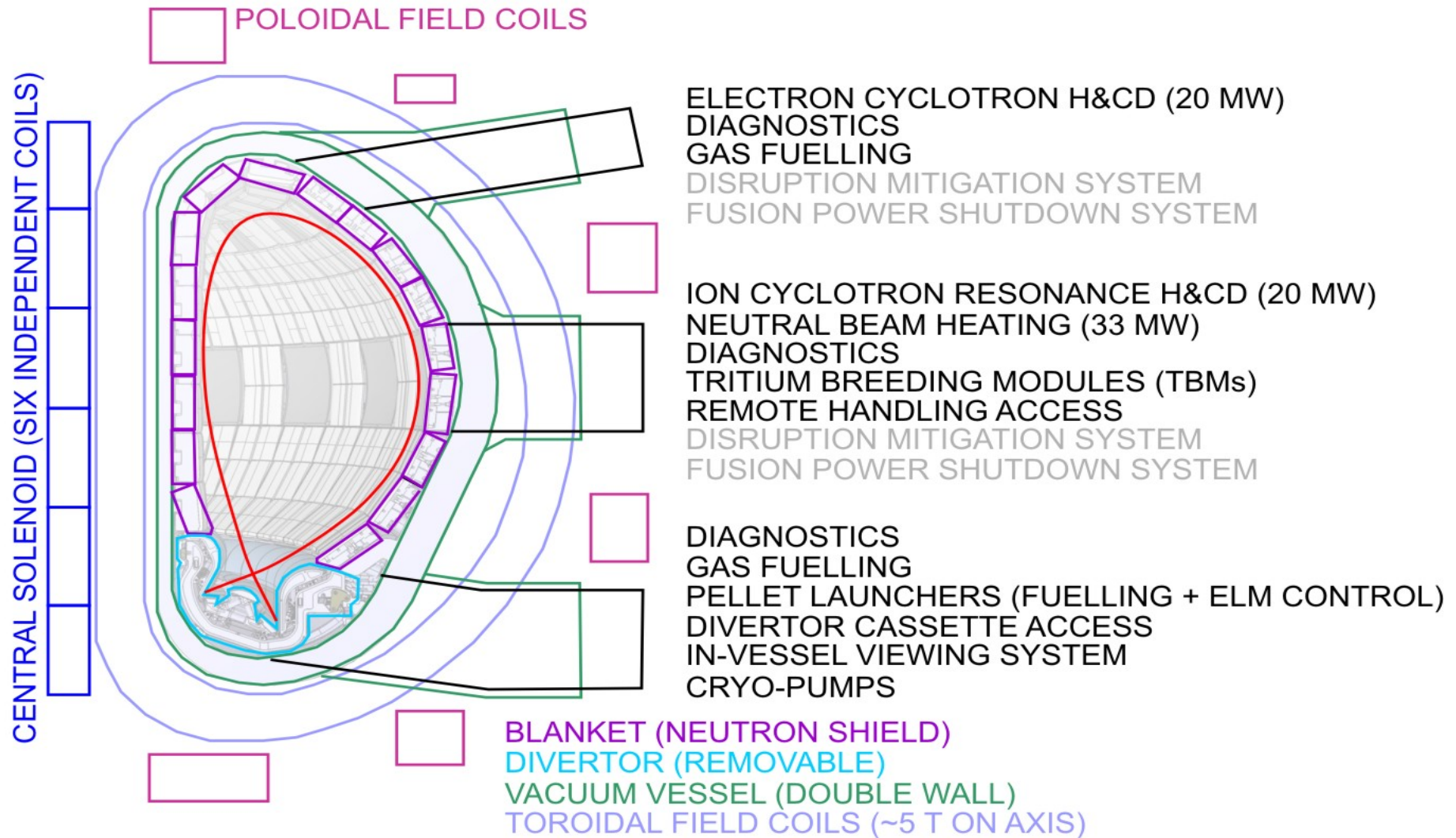


- **On November 10, 2012, the French government published Decree 2012-2048 formally authorizing the creation of the ITER Nuclear Facility**

Distribution of component manufacturing across partners: ITER is shared intellectual property

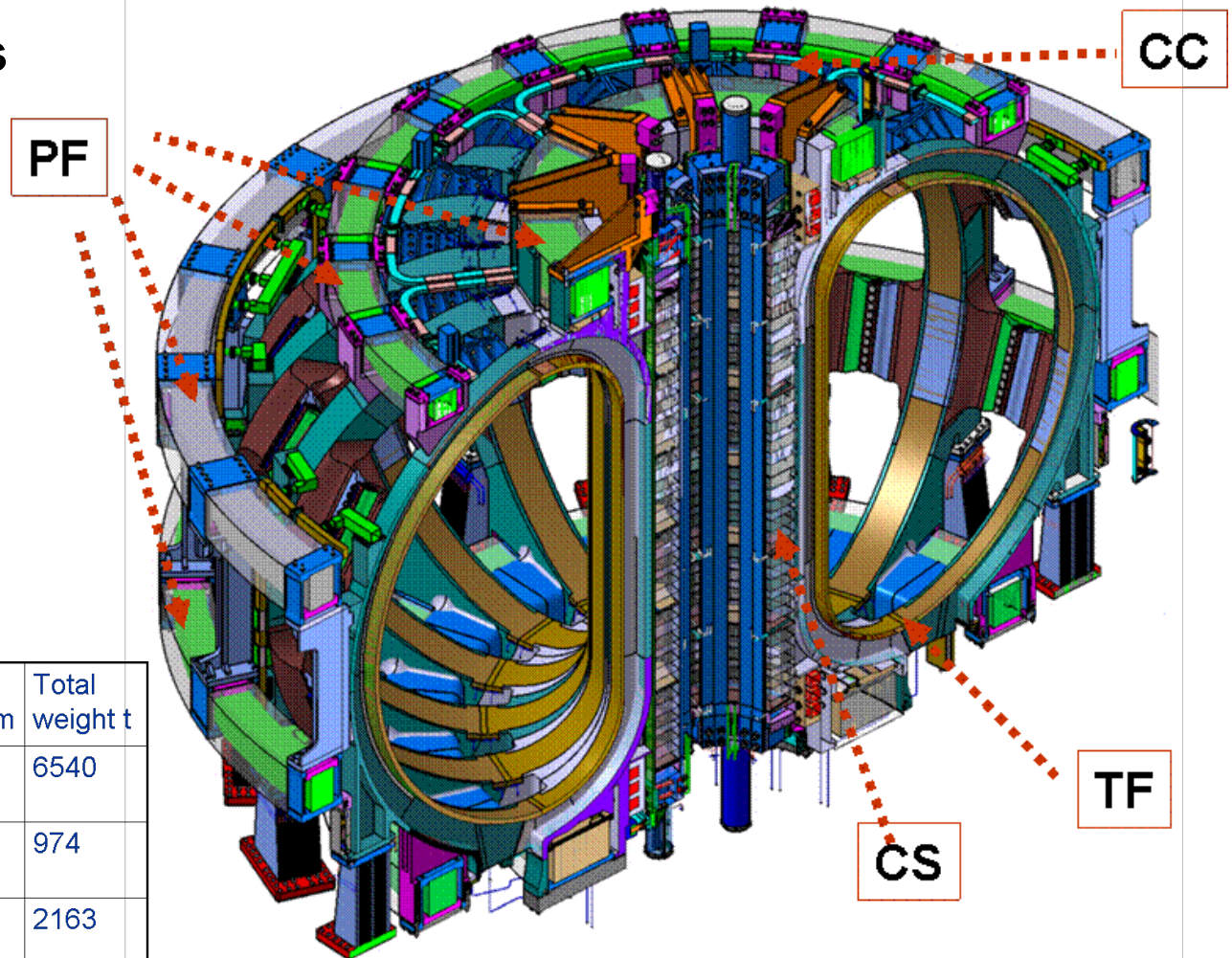


The major components include the vacuum vessel, the magnetic coils, and the heating systems



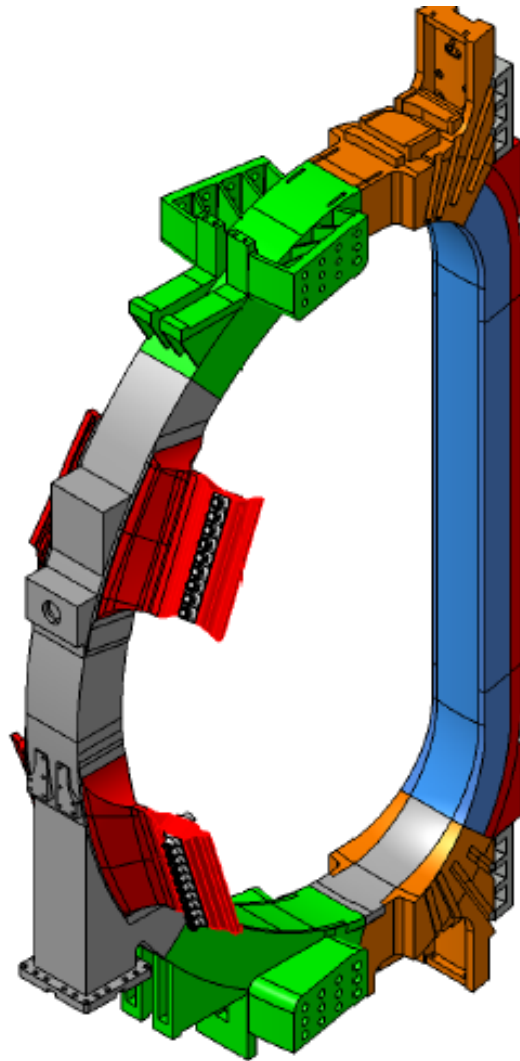
ITER consists of 48 superconducting magnetic coils

- 18 toroidal field coils
- 6 central solenoid modules
- 6 poloidal field coils
- 9 pairs of correction coils



System	Energy GJ	Peak Field	Total MAT	Cond length km	Total weight t
Toroidal Field TF	41	11.8	164	82.2	6540
Central Solenoid	6.4	13.0	147	35.6	974
Poloidal Field PF	4	6.0	58.2	61.4	2163
Correction Coils CC	-	4.2	3.6	8.2	85

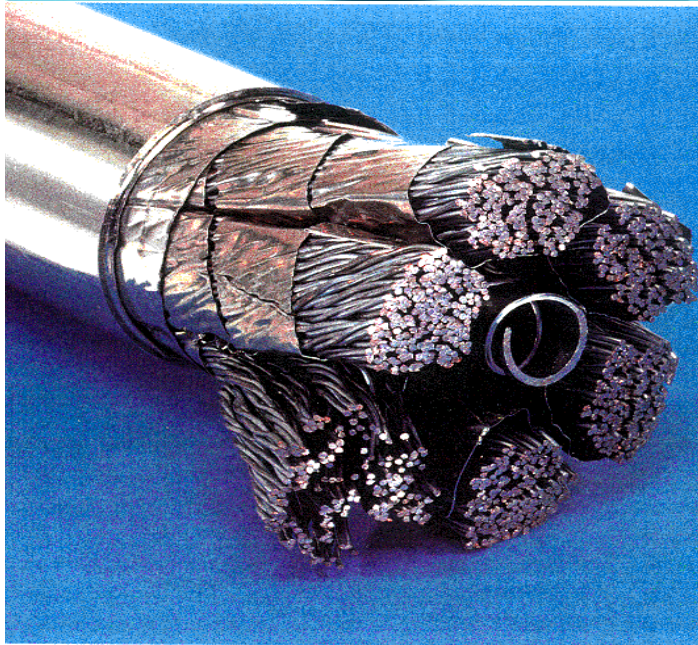
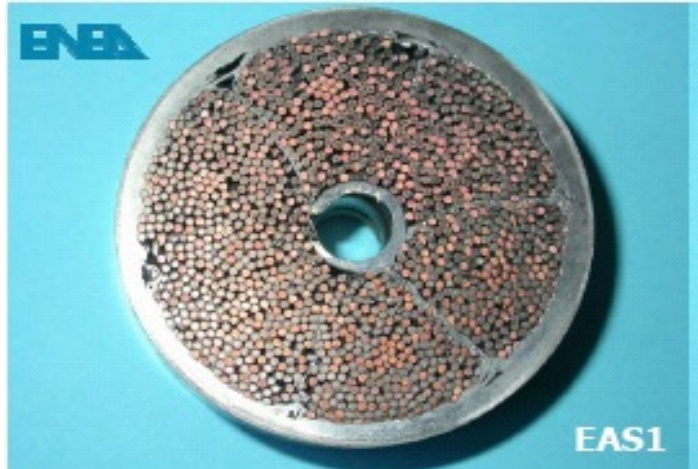
Each toroidal field coils has a total weight of approximately 360 tons



Boeing 747-300: (maximum takeoff weight
~377 t)

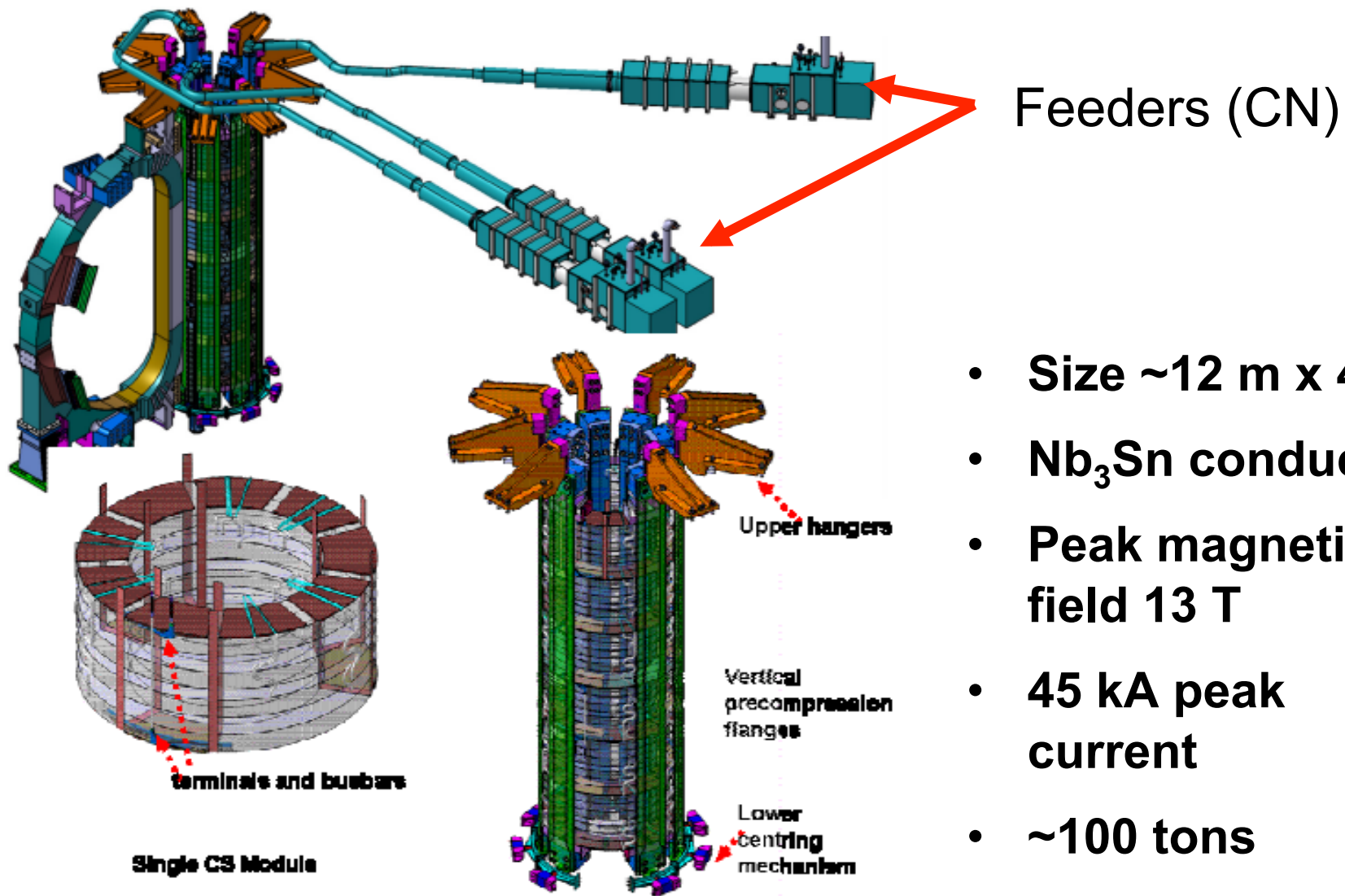
- **16 x 9 m, ~360 t**
- **Fabrication in the EU (Karlsruhe) and Japan (Toshiba): 18 coils**

An unprecedented amount of superconductors is needed for ITER



- **~90 km / 450 tons of Nb₃Sn**
- **~ 150,000 km of strand**
- **Operation at ~5 K**
- **Peak magnetic field 11.8 T**
- **68 kA peak current in coils**
- **Manufacture in EU, Japan, Russian Federation, China, Korea and US**

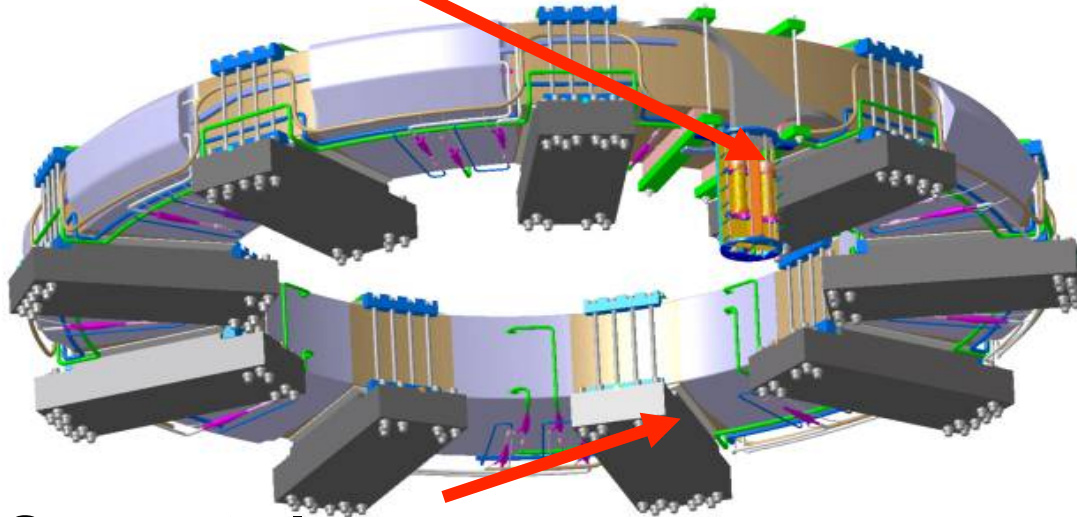
The central solenoid coil stack consists of 6 independently powered modules



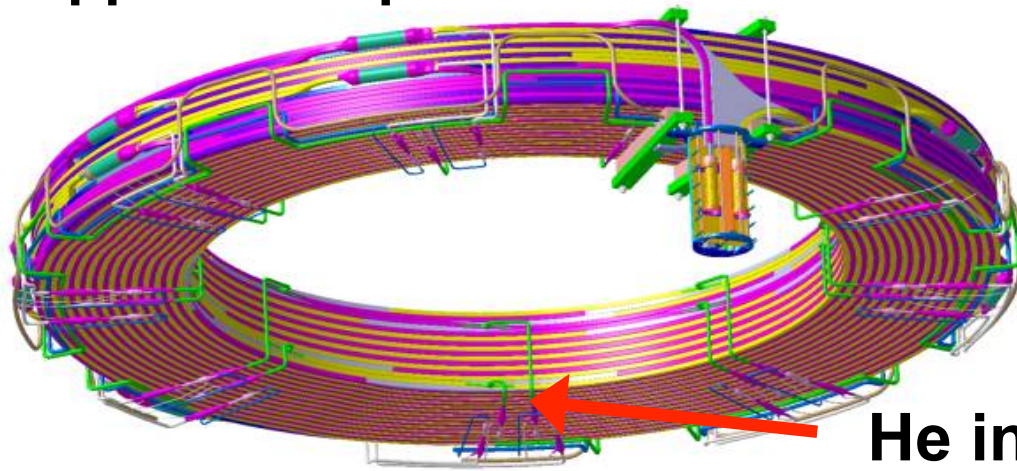
- Size ~12 m x 4 m
- Nb₃Sn conductor
- Peak magnetic field 13 T
- 45 kA peak current
- ~100 tons

There are 6 poloidal field coils, made of NbTi

Terminals

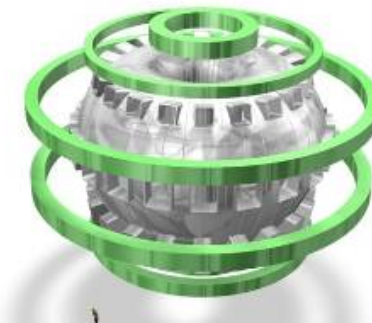


Support clamps

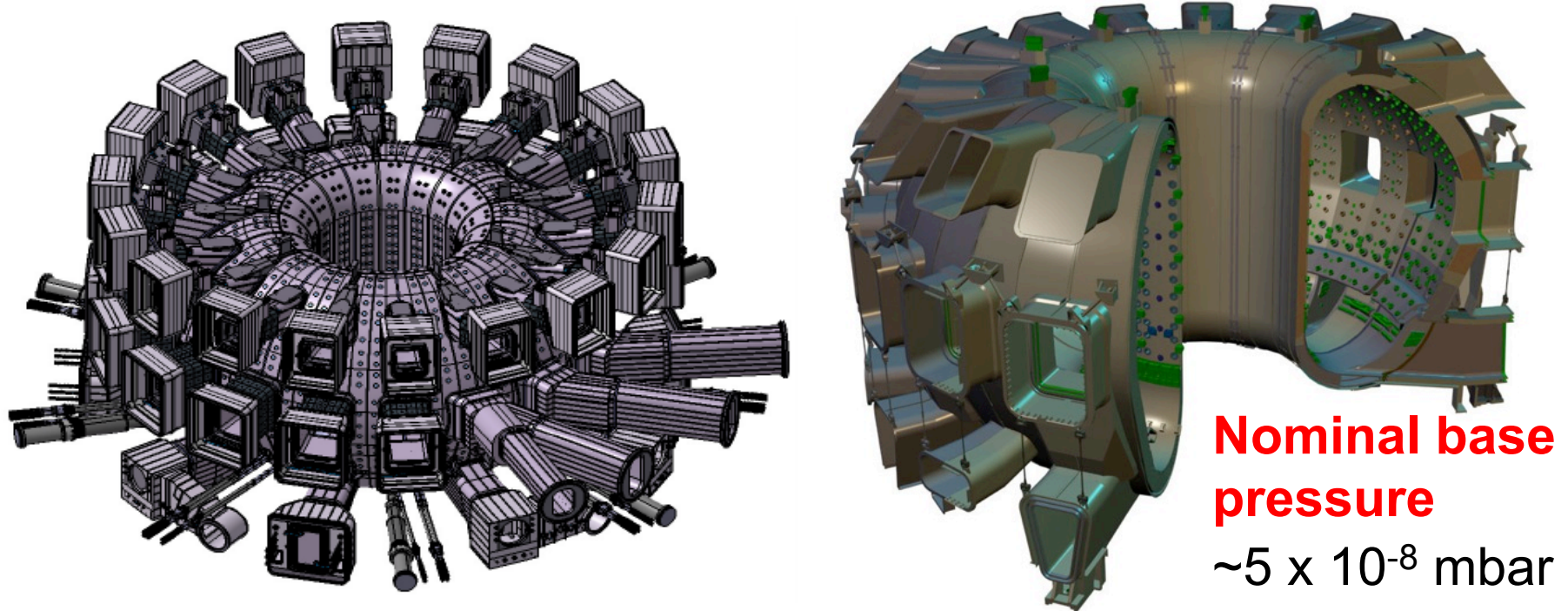


He inlets

- Up to 25 m in diameter
- 6.8 T peak field
- 55 kA peak current
- **Manufactured on-site**

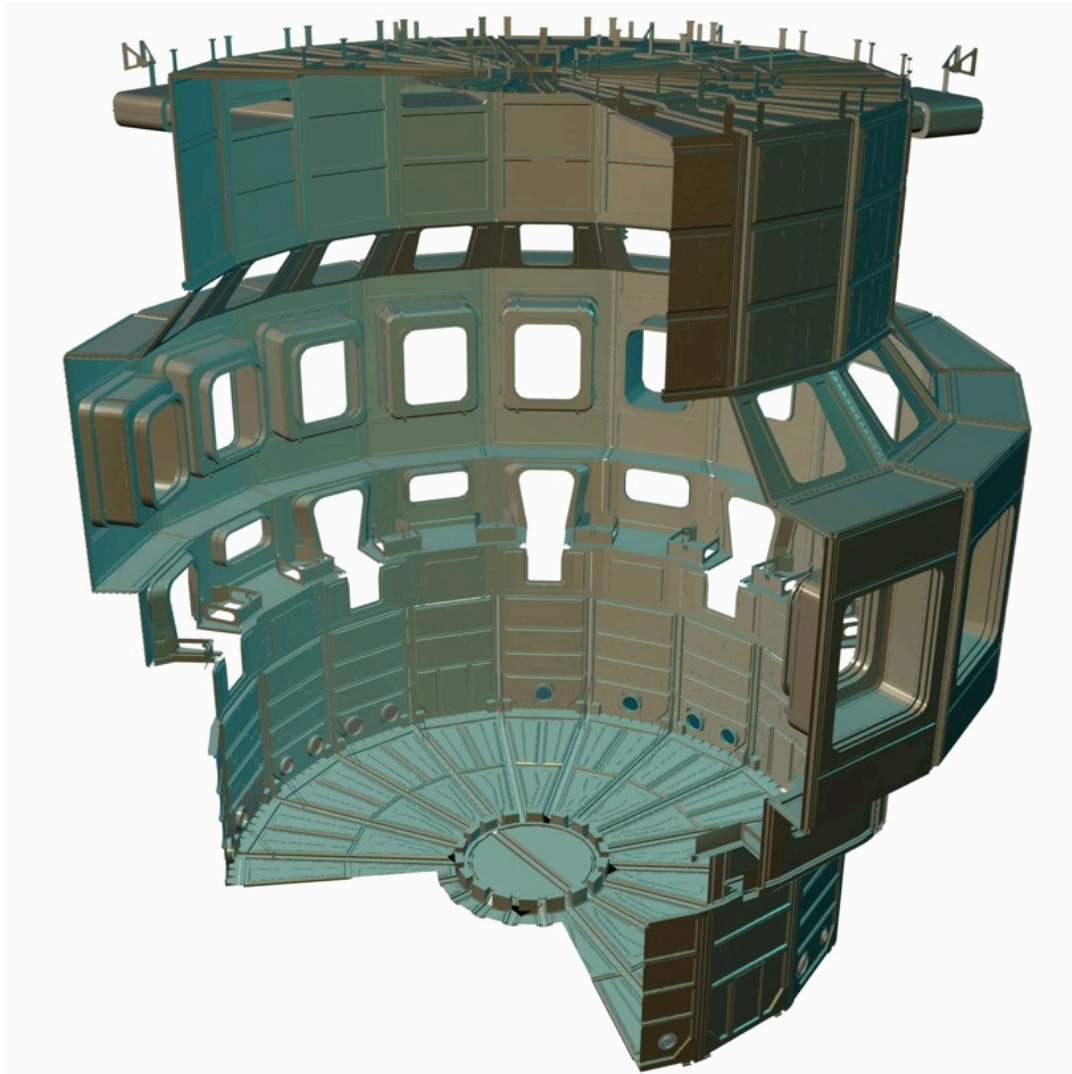


The vacuum vessel is a double-walled, stainless steel structure of about 5300 tons



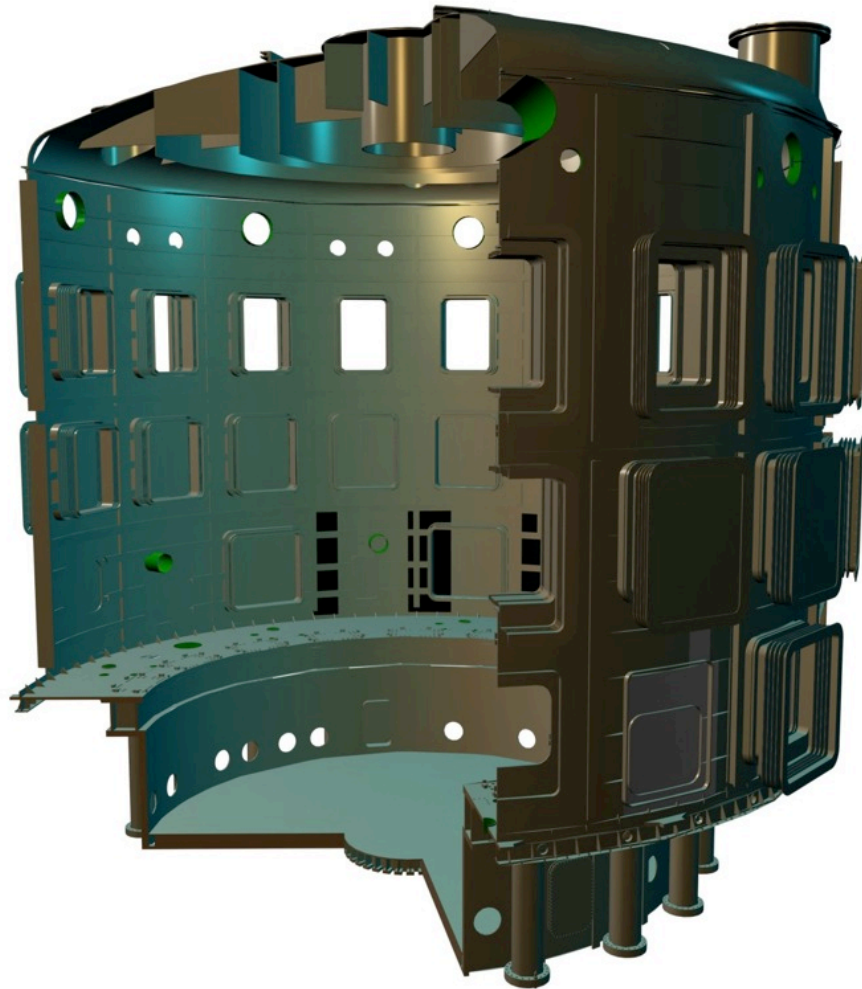
- 19.4 m outer diameter, 11.3 m height
- SS 316 L(N)-IG
- Primary tritium containment barrier, bakeable to 200° C

The main inner heat shield provides a barrier for thermal loads to the superconducting coils (4.5 K)



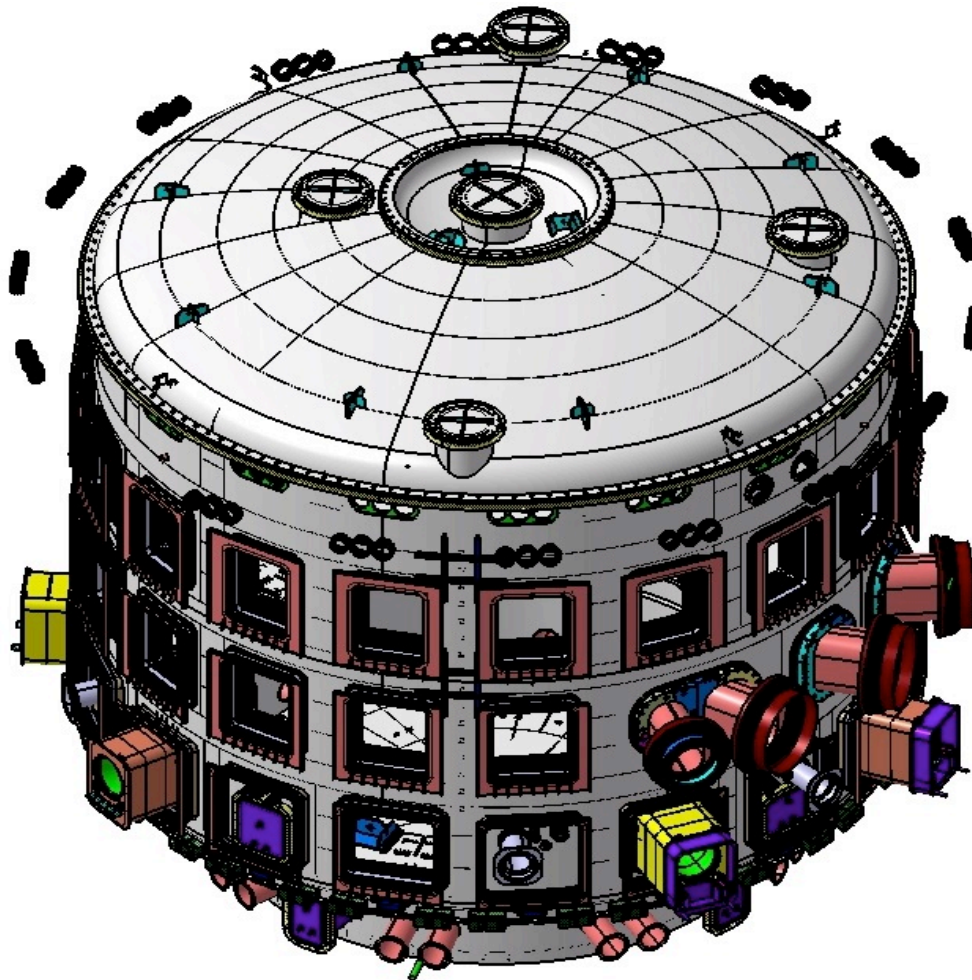
- Operates at 80 K (gaseous He in cooling pipes)
- Stainless steel panels are silver coated to reduce emissivity
- Total mass ~ 1000 t
- Smaller shield isolates TF coils from vacuum vessel

The cryostat provides the vacuum insulation for the superconducting coils

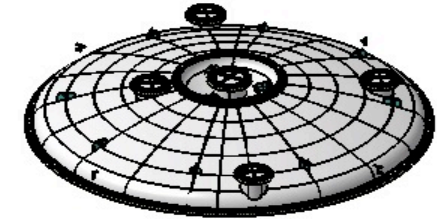


- 29.4 m in diameter, ~29 m in height
- Total mass ~ 3500 t
- Base pressure $< 10^{-4}$ mbar
- Stainless steel 40 – 180 mm thick

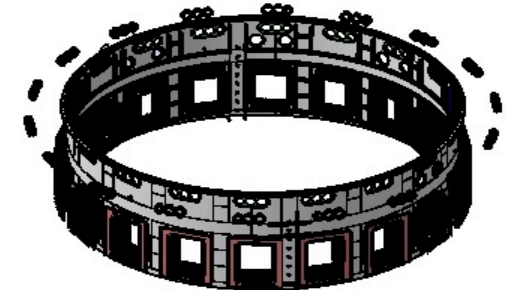
The cryostat provides the vacuum insulation for the superconducting coils



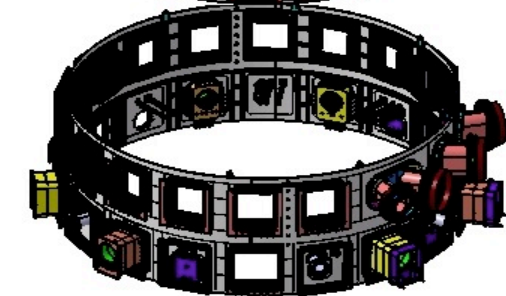
Top lid



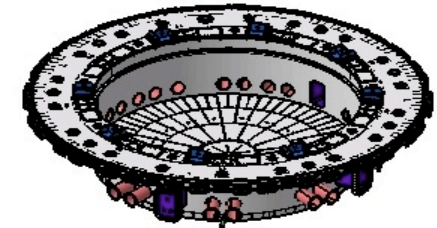
Upper cylinder



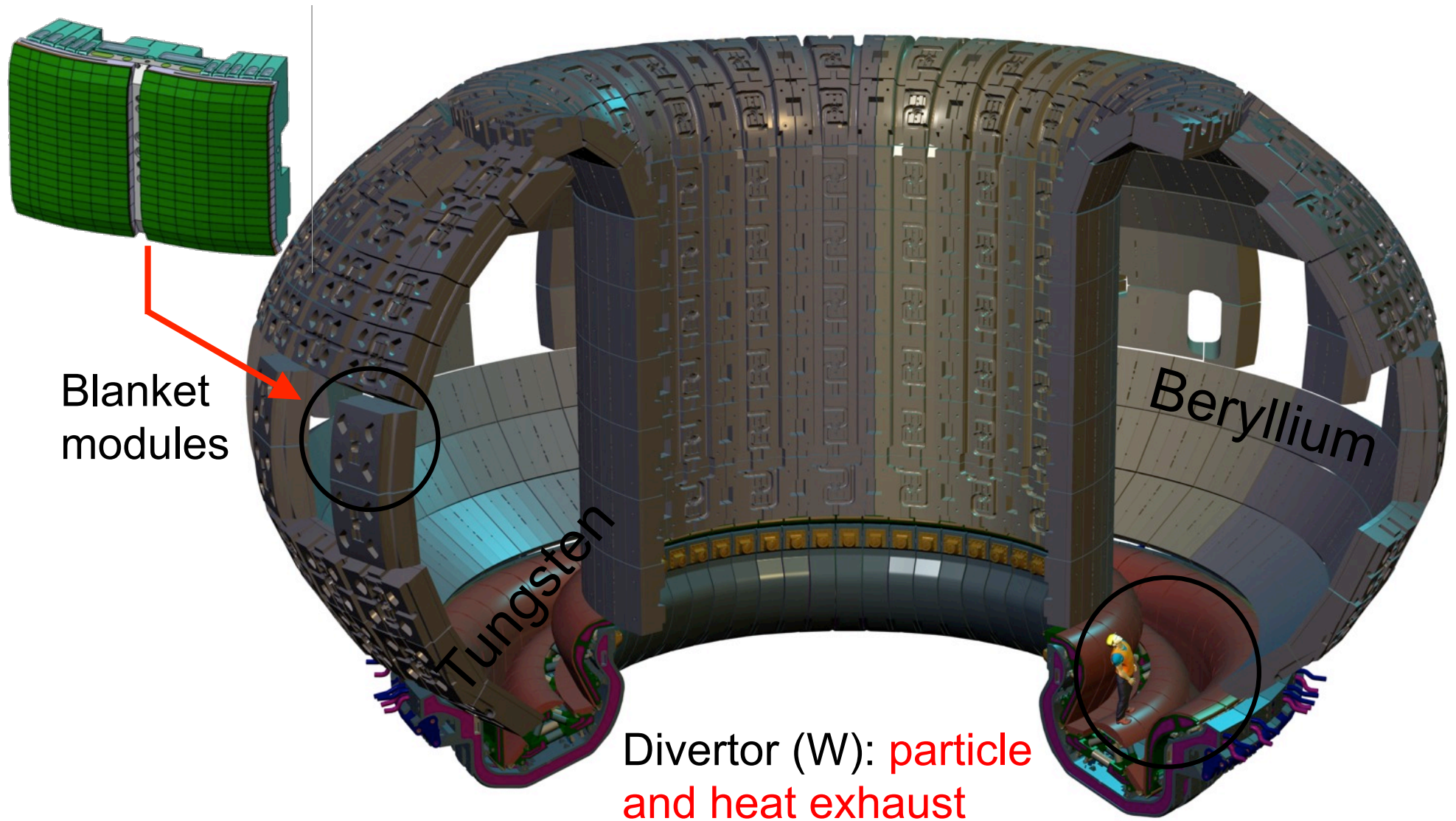
Lower cylinder



Base section



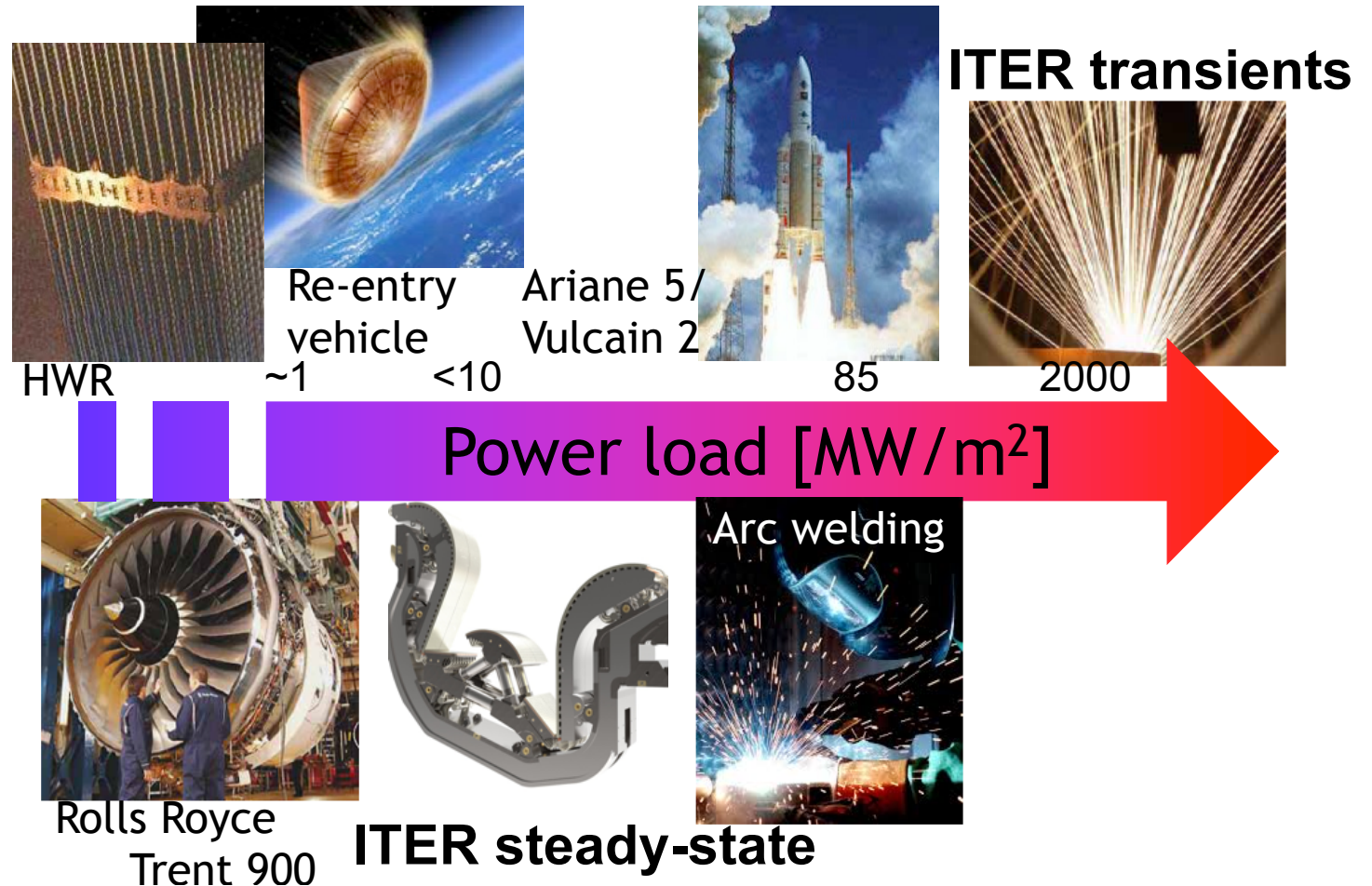
The first wall, or blanket, provides the interface of the machine to the (burning!) plasma



The power loads to the plasma-facing components in ITER exceed those of space rocket launches

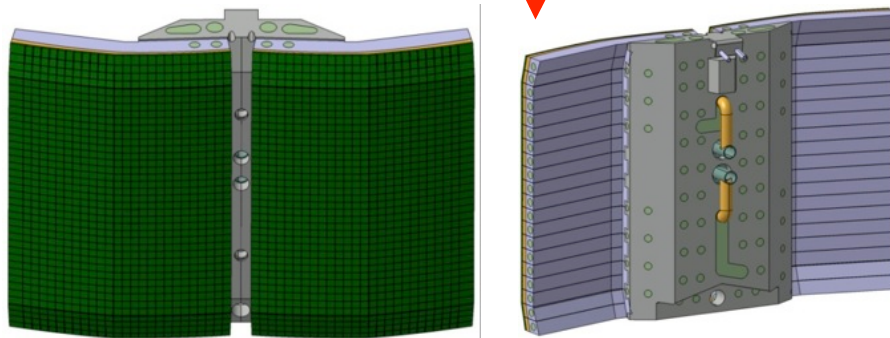
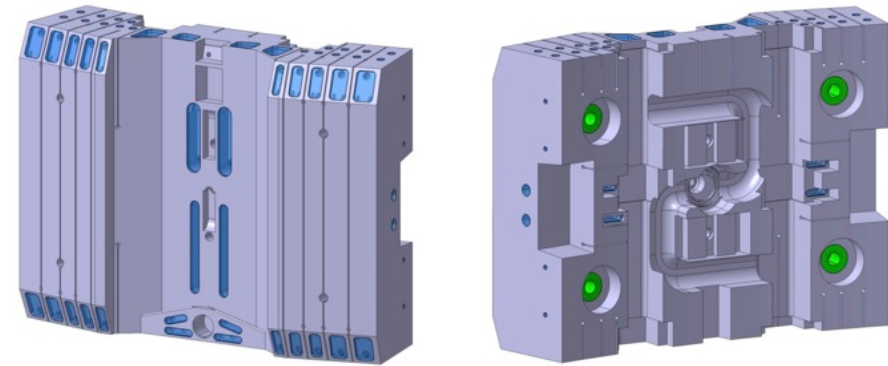
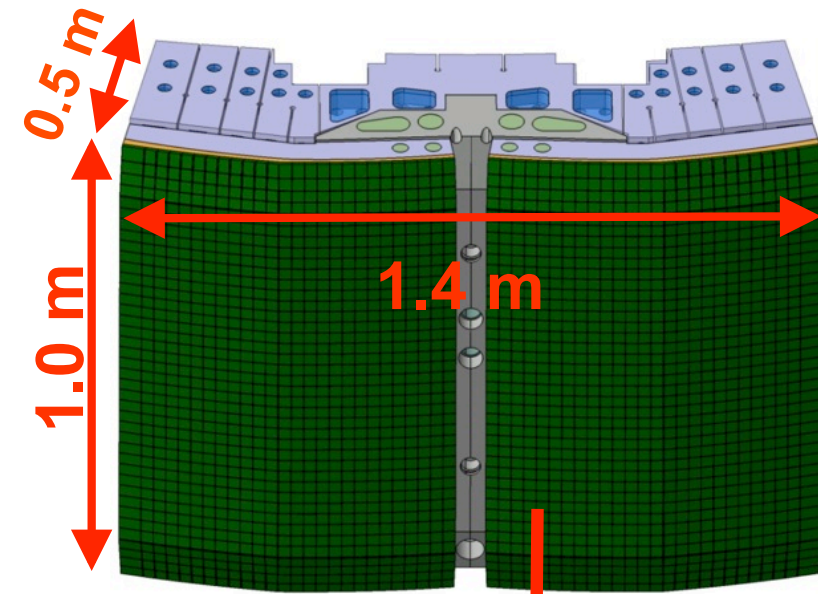
The challenge is not small!

The consequences of failing to protect in-vessel components are big—components are actively cooled (water) and replacement time is months



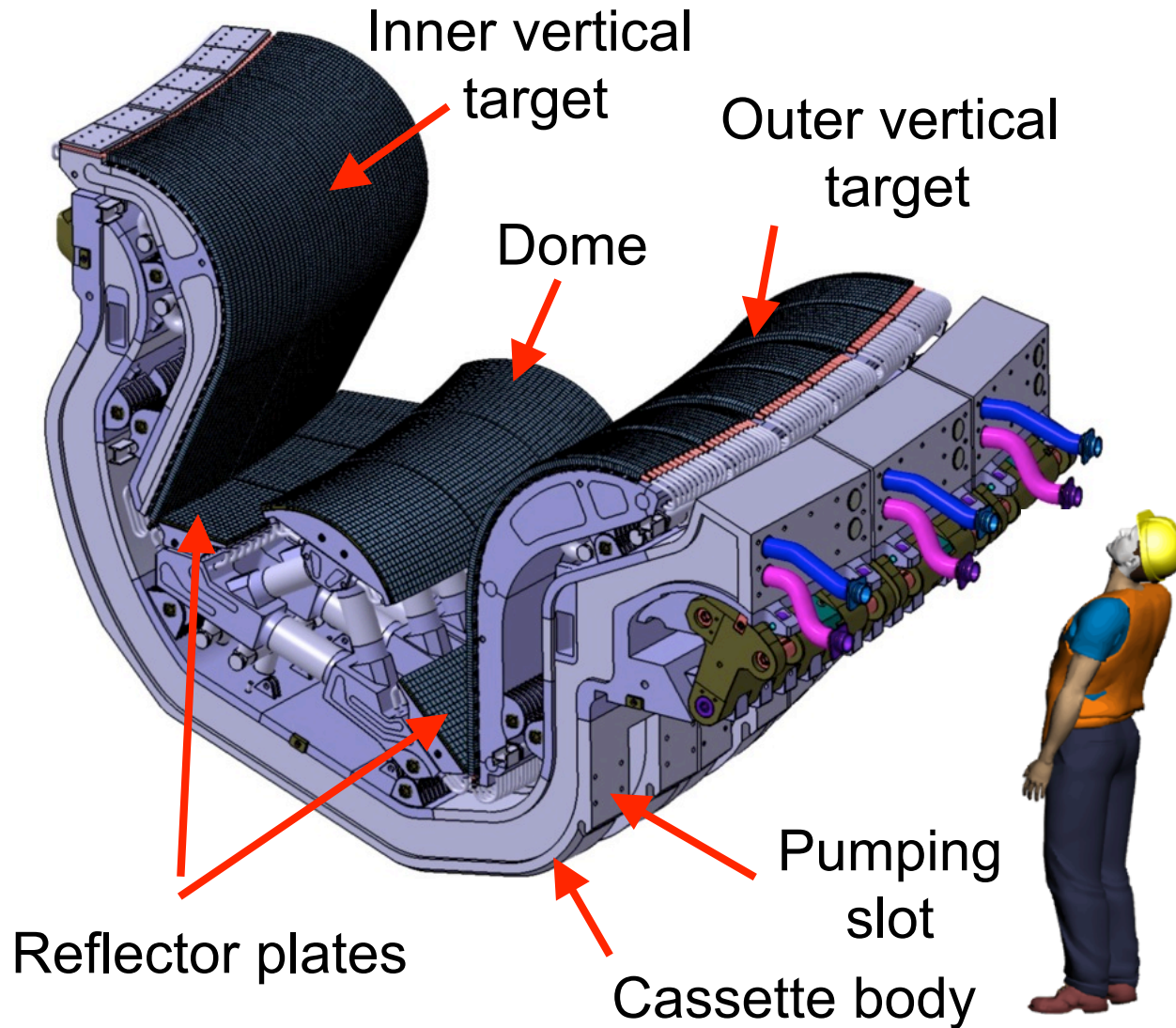
Luce, APS-DPP Oct 2018

The first-wall blanket modules provide both neutron and plasma shielding



- **Semi-permanent massive shield block for neutrons (3.5 t)**
- **Shaped first-wall panels armored with Be**
- **Total number of blanket modules: 440 (1800 t)**

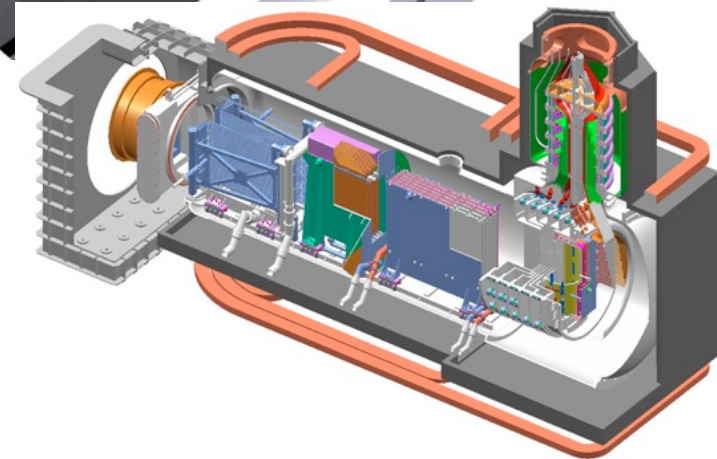
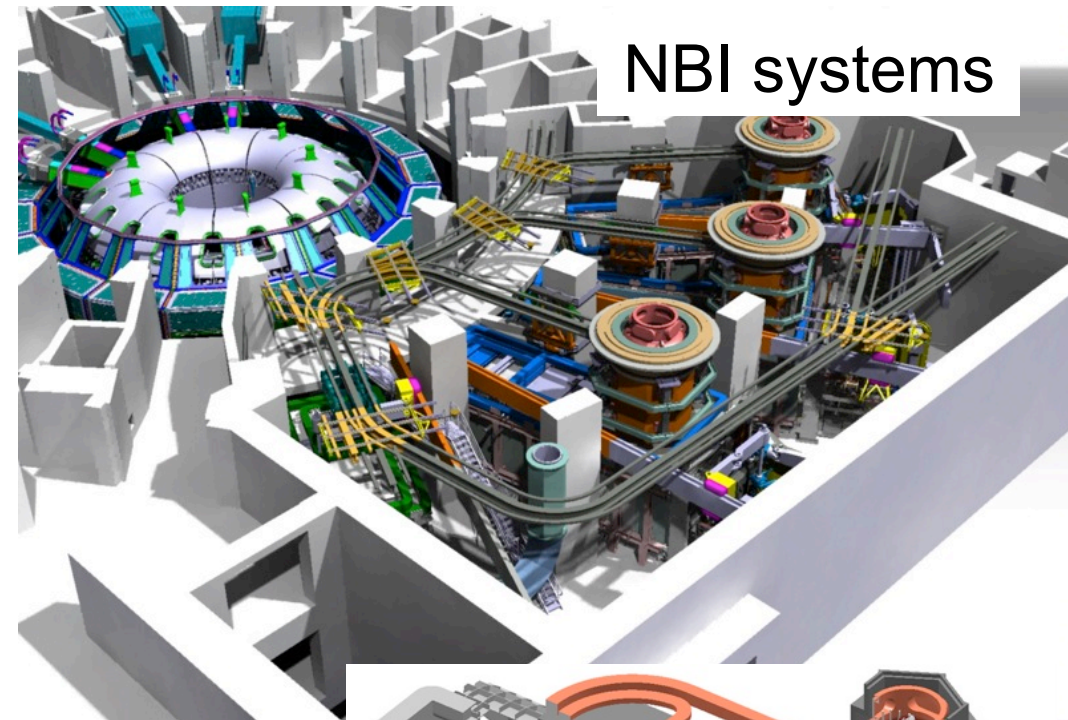
The divertor consists of 54 cassettes (~ 8.7 t each) bakeable to 350° C



- 4320 actively cooled heat flux components
- Divertor PFCs are made of tungsten
- Removable cassettes

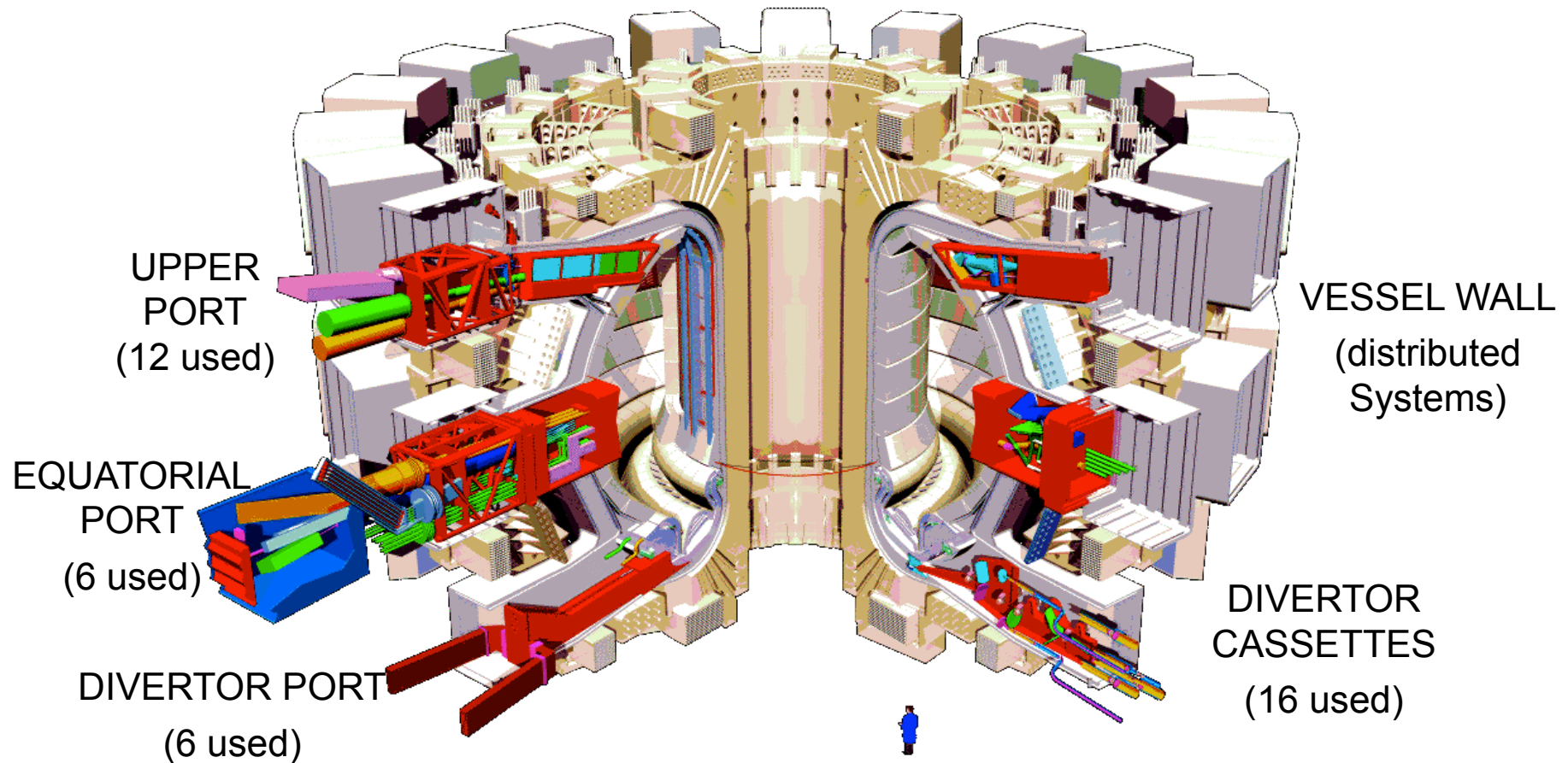
Auxiliary power by means of neutral beams and radio frequency heating of 50 MW is foreseen

System	Power
NBI -ve ion, 1 MeV	33 MW
ECH & CD 170 GHz	20 MW
ICRH & CD 40 – 55 MHz	20 MW



- P_{aux} for $Q_{\text{DT}} = 10$ about 40-50 MW
- **Upscale of known technology**
- **Modular for upgrades**

About 40 major diagnostic systems will be installed for machine protection, control and physics



- **Daily (raw) data output is expected to reach petabytes**

About 20 parameters will be controlled simultaneously on time scales from 1 ms to seconds

Plasma Control Matrix	ACTUATORS															
	Actuators	Electron Cyclotron	Ion Cyclotron	Neutral Beam	Toroidal Field	Poloidal Fields	Correction Coils	Internal VS Coils	ELM Control Coils	Fuelling Gas Injectio	Impurity Gas Injectio	Fuelling Pellet Injectio	ELM Pacemaking Pe	Impurity Pellet Injectio	Vacuum Pumping	Disruption Mitigation System
Control Parameter Sets																
Wall conditioning & Tritium removal																
Error fields																
Plasma breakdown																
Plasma c																
Plasma s																
Plasma p																
Internal																
ICRF cou																
Divertor																
Divertor																
Divertor																
ELM freq																
Electron																
Fuelling																
Impurity																
Helium f																
Core D/T																
Fusion p																
Plasma s																
Beta toro																
Plasma rotation																
Current density profile																
Core radiation																
Sawtooth period/amplitude																
NTM control																
RWM control																
Disruption control																
Controlled plasma shutdown																
Runaway electron control																
Disruption mitigation																

- Extensive control matrix, requires state-of-the-art schemes

Summary

- **ITER is a multi-national collaboration to construct the first burning-plasma fusion reactor based on the tokamak concept**
 - **Significant challenges in physics and technology, and the integration of both, licensing of project**
- **Construction of buildings and components (off-site) commenced in 2009 ⇒ by November 2020, 75% of the civil engineering work was completed ⇒ construction of actual tokamak started**
- **The first plasma (non-nuclear, hydrogen or helium) is anticipated for 2025, the first D-T $Q=10$ plasma in 2035**

ITER is moving forward

ITER is moving forward!

Thank you for your attention
<http://www.iter.org>